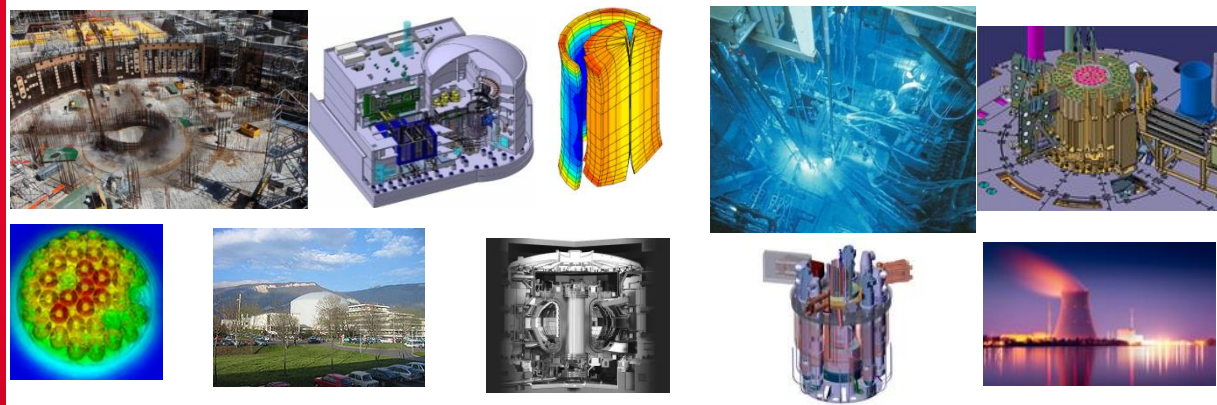


DE LA RECHERCHE À L'INDUSTRIE

cea



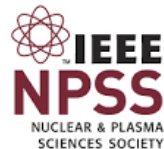
Radiation detection and measurement for non-destructive characterization and control in nuclear media.

Abdallah Lyoussi
CEA/DES/IRESNE

Senior Fellow

IEEE Distinguish Lecturer

Abdallah.lyoussi@cea.fr



Rabat Educom Instrumentation Summer School; Univ Of Rabat, July 1st-11th 2024

- ❑ INTRODUCTION

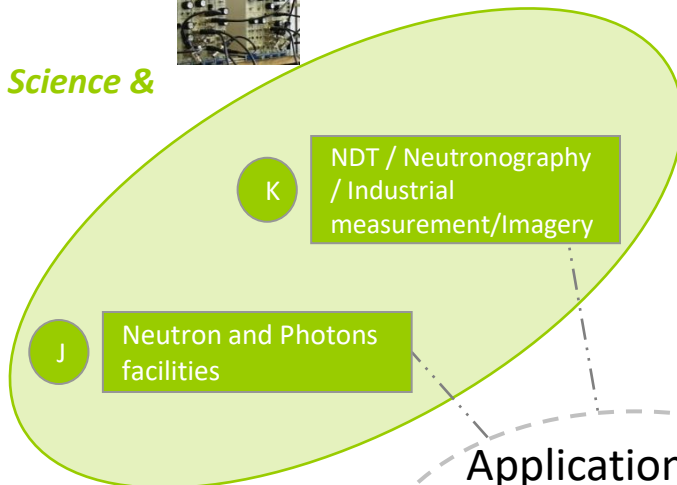
- ❑ Specific features, constraints and requirements

- ❑ Examples of radiation measurements and associated instrumentation for :
 - In Pile Measurements
 - Radioactive waste characterization and control

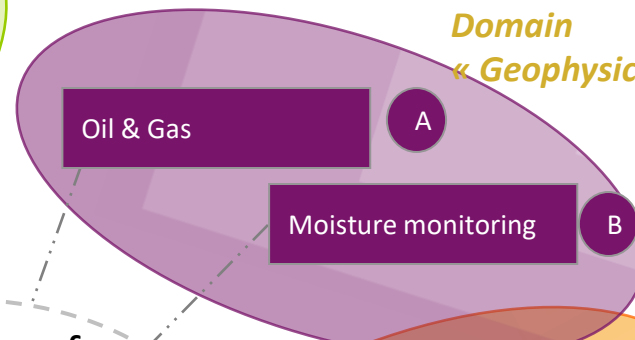
- ❑ Conclusions and prospects

RADIATION DETECTION & MEASUREMENTS: APPLICATION DOMAINS / SUB-DOMAINS

Domain
« *Materials Science & Industry* »

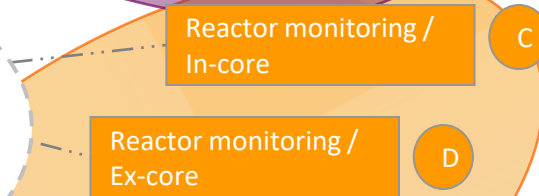
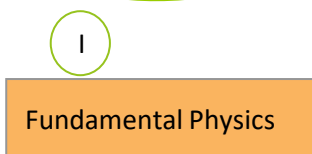


Domain
« *Geophysics* »



Applications of Instrumentation and radiation detection systems

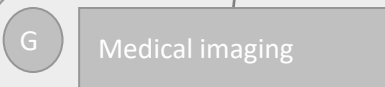
Domain
« *F. Physics & Space* »



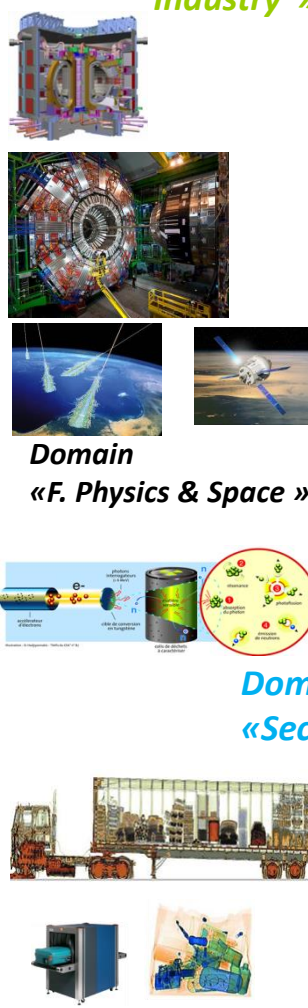
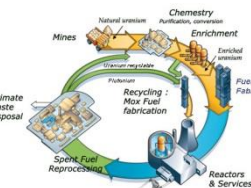
Domain
« *Security* »



Domain
« *Nuclear* »



Domain
« *Medical* »



2 Main Families

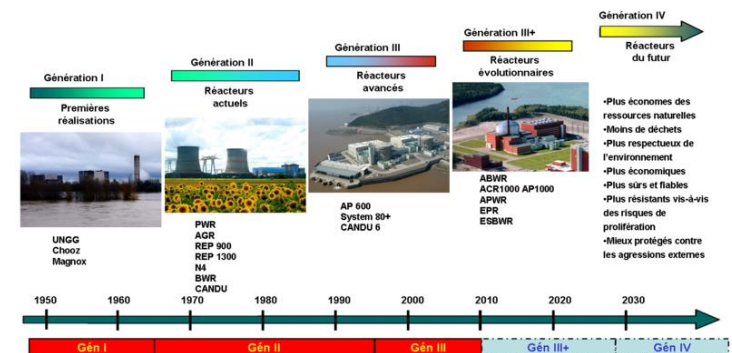


Experimental and Research Reactors :

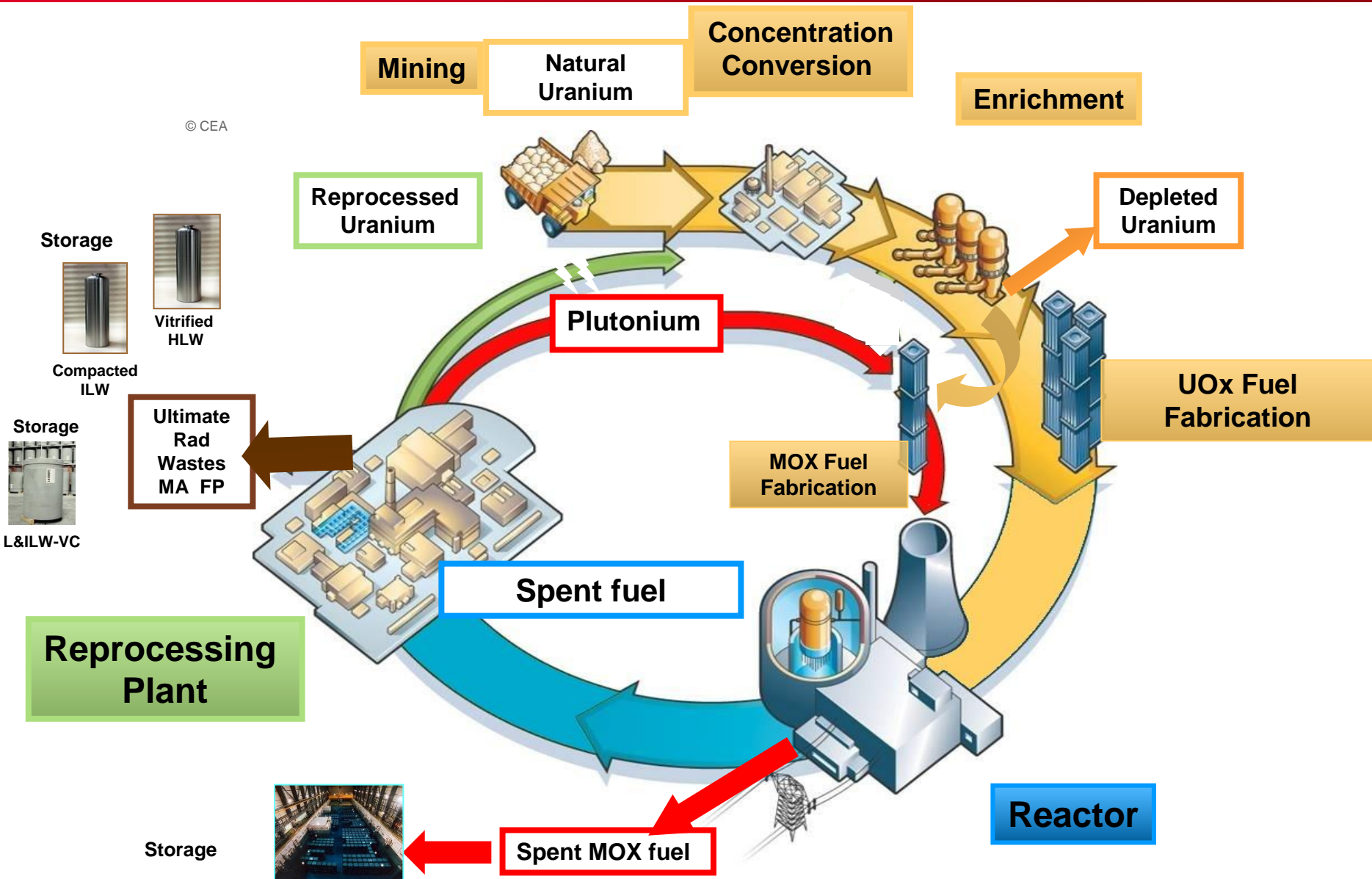
- Zero Power Reactors ZPR (Eole, Minerve, Masurca, LR0...)
- Material Research Reactors MTRs (Osiris, JHR, BR2, Maria, ATR...)
- Research Reactors (ILL, Orphée, TRIGA)
- Safety Study Reactors (Cabri, Treat...)
- Reactors for special applications (RES)
- Research and Education Reactors (Ulysse, Minerve, ISIS, TRIGAs...)

Nuclear Power Reactors NPPs :

- GEN II (REP, REB ...)
- GEN III et III+ (AP600, EPR, AP1000 ...)
- SMR, AMR
- GEN IV (SFR...)



Nuclear Fuel Cycle



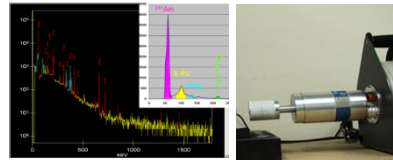
Nuclear Fuel Cycle :

Instrumentation & measurement are key aspects for control & characterization

Measurements and Specific Controls are Strongly needed and necessary at each step /sub-step of Nuclear Fuel Cycle.



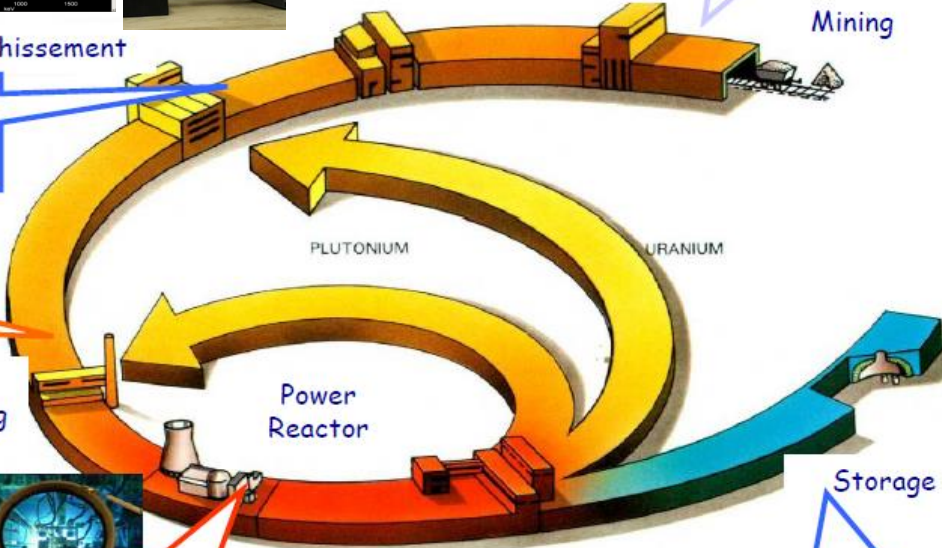
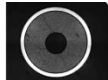
«From Mining to Waste Storage : Main Fuel Cycle Steps»



Enrichissement

Identification and Characterization of final product

- Mechanical properties of Fuel
- Advanced Characterization of Fuel
- Non Destructive Assays



Measurement in Experimental Reactor under representative conditions.

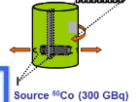
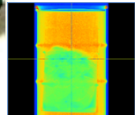


Research Reactor

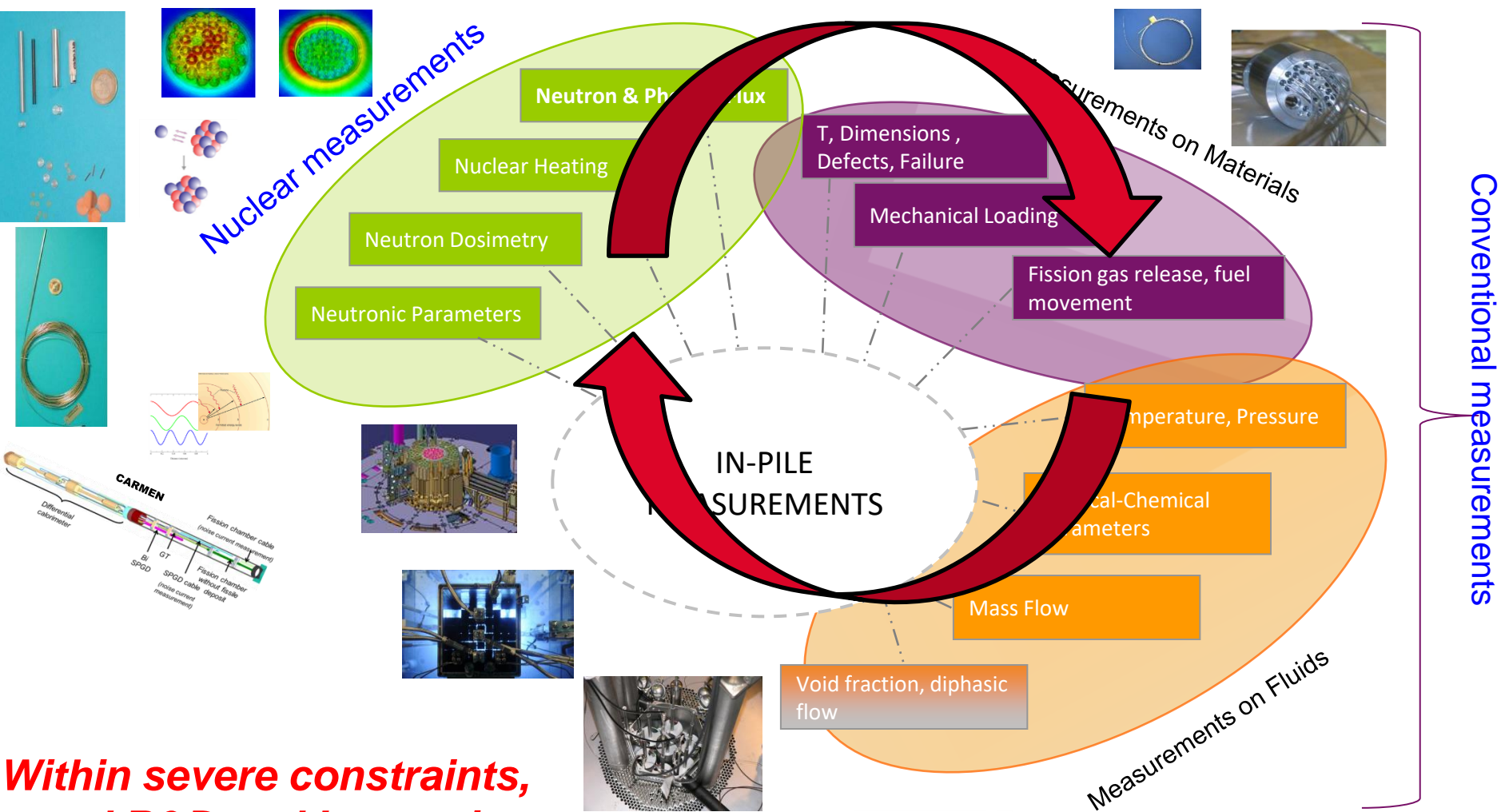
Monitoring and rad. protection : Control, NDA

Radioactive Waste Characterization & Management ...

- Chemical & Physical Characterization
- Control of U-Pu
- Working Control



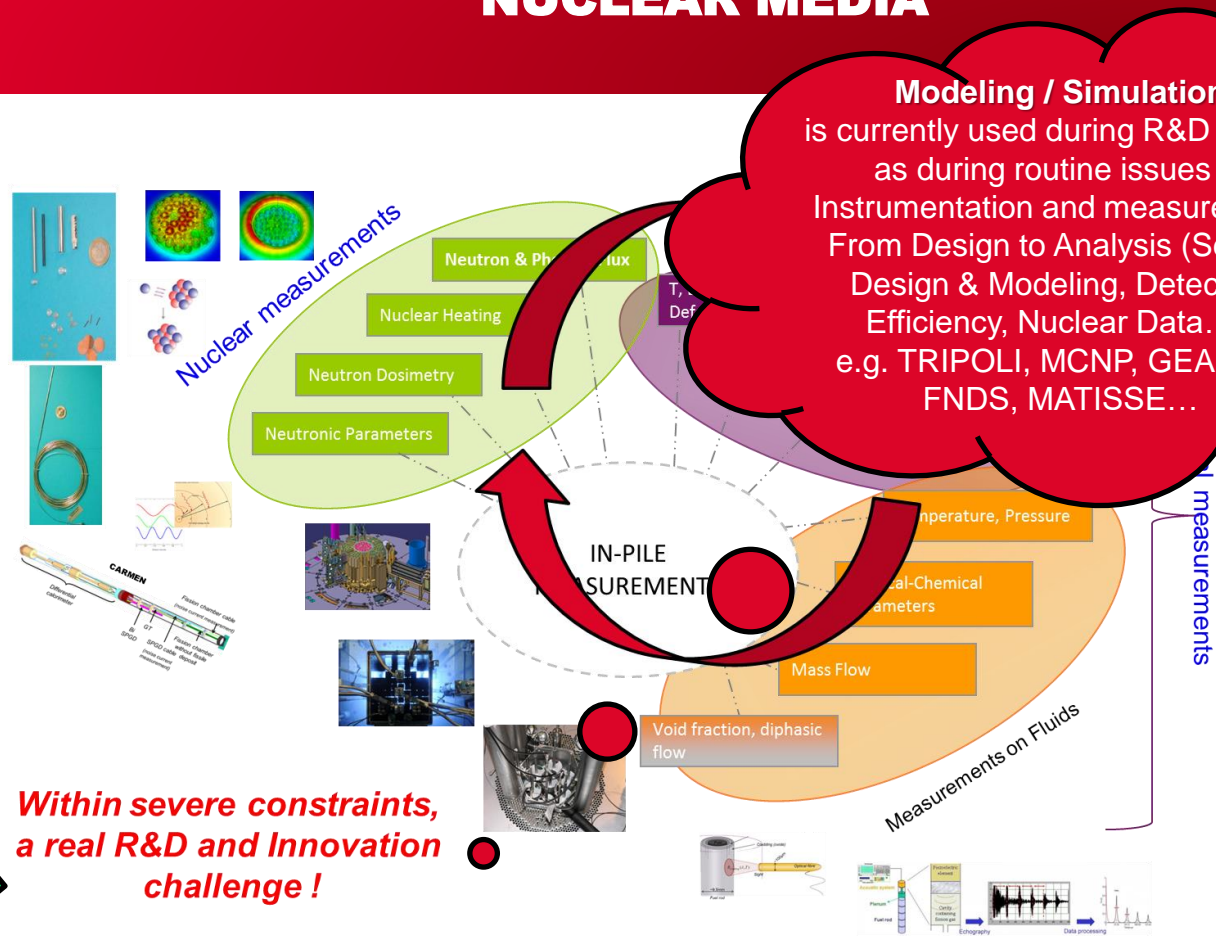
Main in-pile measurements



Within severe constraints, a real R&D and Innovation challenge !



INSTRUMENTATION & MEASUREMENTS IN NUCLEAR MEDIA



Within severe constraints, a real R&D and Innovation challenge !

Also :
(D/D, RW, NDA, Safeguards)

Dosimetry, Gamma and X-ray spectrometry, Gamma and X Imaging, Neutron Imaging, Alpha radiography, Beta spectrometry, passive & active neutron measurement, PNAA, DNAA, Photon interrogation.

CHALLENGES FOR INSTRUMENTATION & MEASUREMENTS IN NUCLEAR ENVIRONMENTS

Sensitivity & Selectivity

Reliable

impossible or difficult maintenance on irradiated objects

Accurate despite a very severe environment

to follow modelling progress; ex: μm dimensional measurements, $\Delta T < 5^\circ\text{C}$

Miniature

narrow location to get maximal neutron flux: few mm available

Corrosion resistant

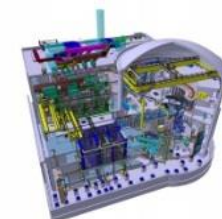
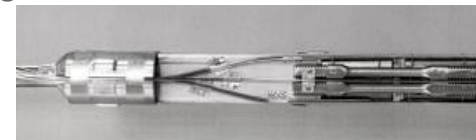
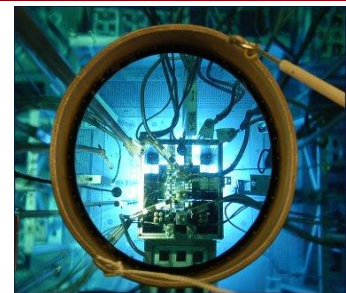
operation in pressurized water, high temperature gas, liquid metals...

High temperature resistant

> 300°C , up to 1600°C

Neutron / γ "resistant"

dose > 15kGy/s and > 10dpa/y



Extremely Conservative

- ❑ INTRODUCTION
- ❑ Specific features, constraints and requirements
- ❑ Examples of radiation measurements and associated instrumentation for :
 - **In Pile Measurements**
 - Radioactive waste characterization and control
- ❑ Conclusions and prospects

Reminder :

Main Aim of Nuclear Reactor Measurements : To reduce uncertainties

To reduce uncertainties at each step of the process; **from conceptual design to final running of nuclear system(s)**

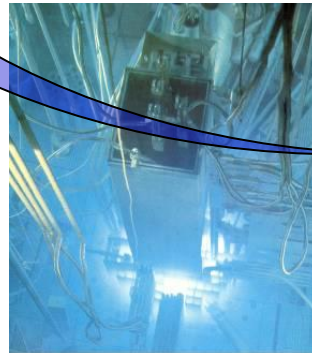
Basic/Fundamental Data

Measurement of Basic Physical Data



Design Tools

Smart/Analytical Experiments
→ Predictive / Calculations
Model Development



Verification

Global/Integral Experiments on ZPR Reactors
→ Check, verify, predictions
Validation of Models/ Calculation schemes



Online Monitoring

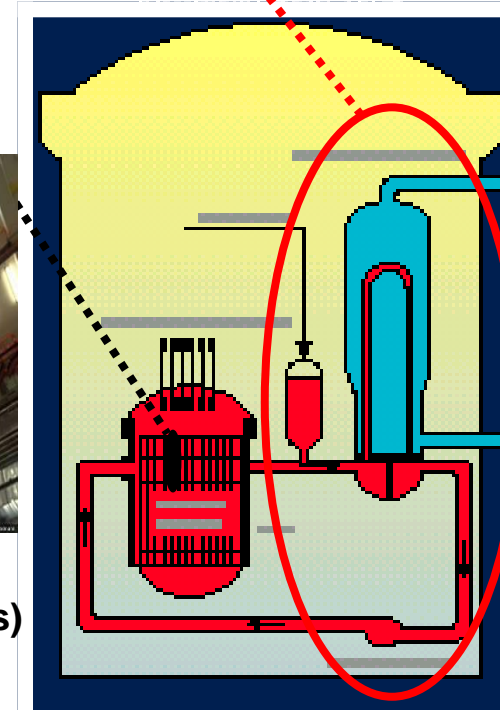
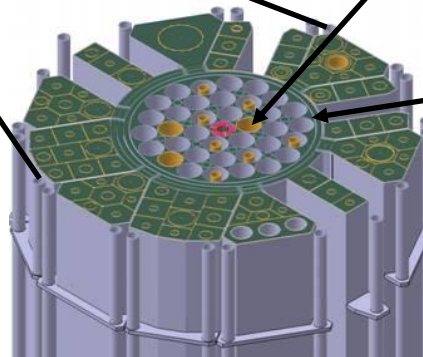
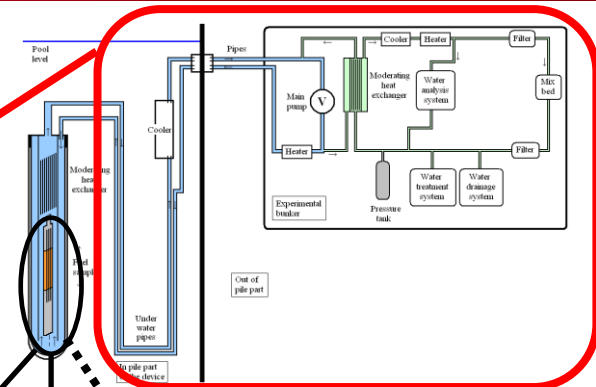
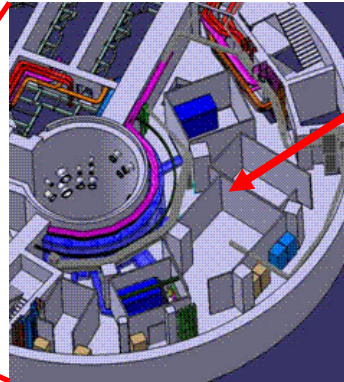
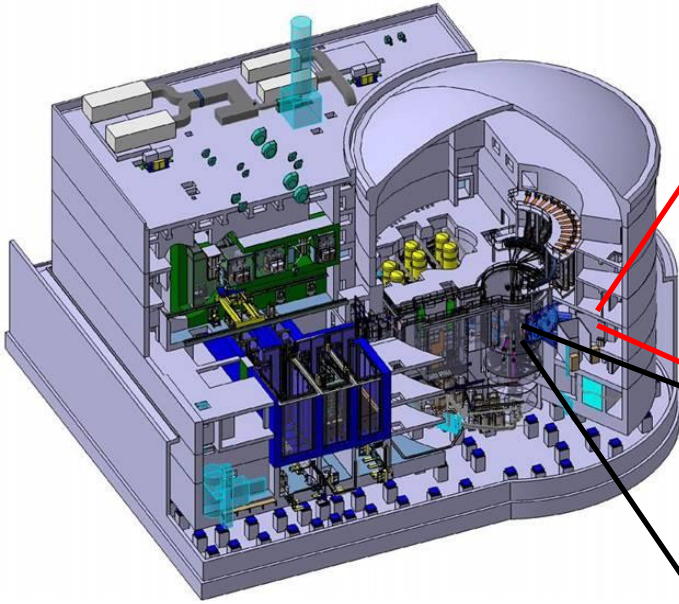
Monitoring & Control in NPP



In Pile Measurements, Qualification and Testing in MTRs



What are the Objectives of a MTR (Material Testing Reactor) ?

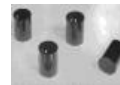


MTR allows to reproduce on a small scale, real power plant conditions and in some cases, more severe conditions for

Material screening (comparison of materials tested under representative conditions)

Material characterization (behaviour of one material in a wide range of operating conditions, up to off-normal and severe conditions)

Fuel element qualification (test of one / several fuel rods (clad+fuel))

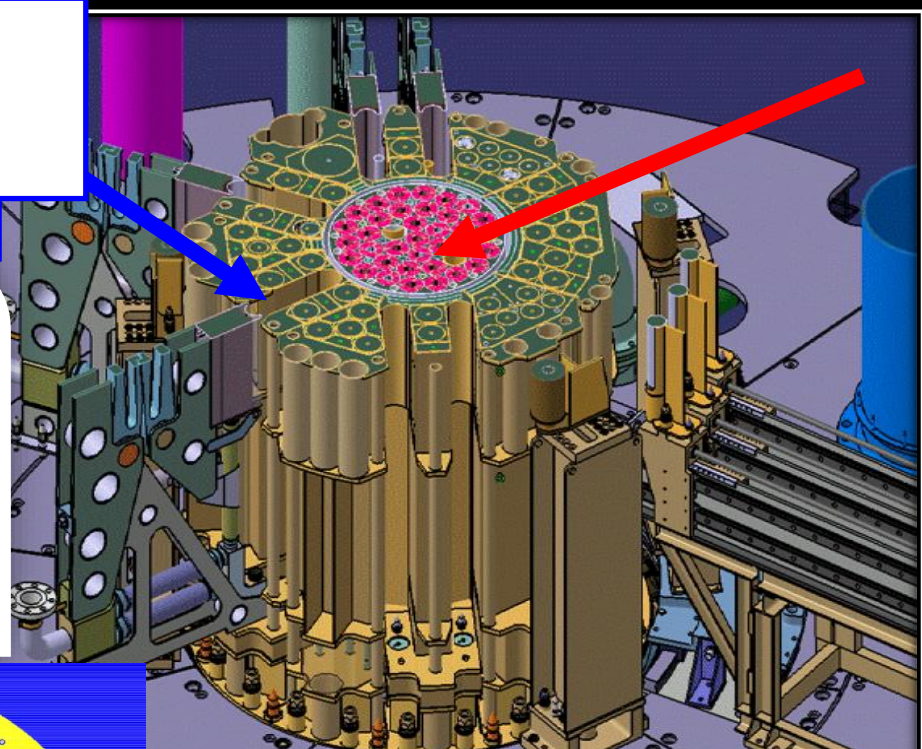
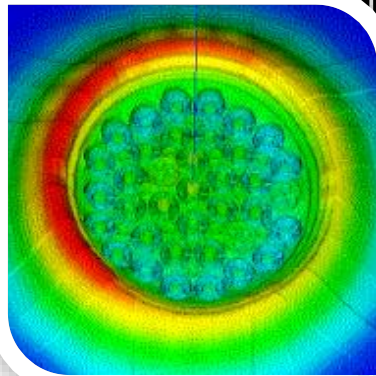


JULES HOROWITZ (JHR) MTR REACTOR

Reflector

$\Phi \geq 5.5 \cdot 10^{14}$ n/cm².s
20 fixed locations
6 mobile locations

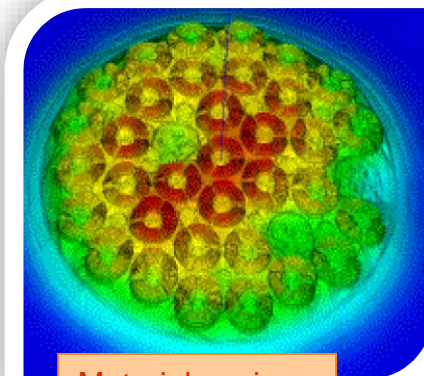
Thermal neutron flux



Core

$\Phi \geq 5.5 \cdot 10^{14}$ n/cm².s > 1 MeV
 $\Phi \geq 10^{15}$ n/cm².s > 0.1 MeV

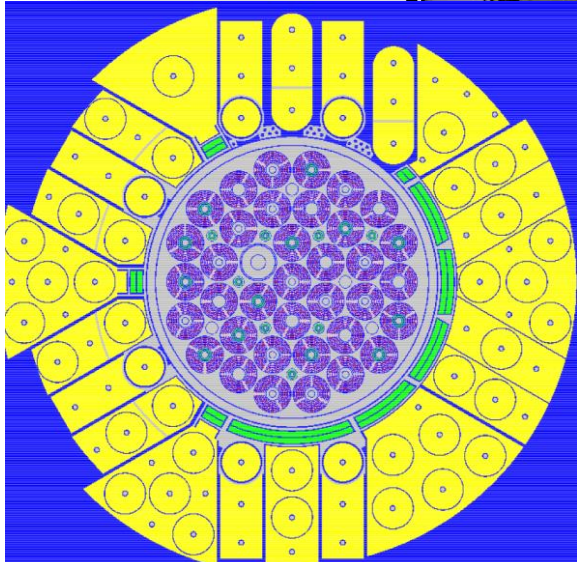
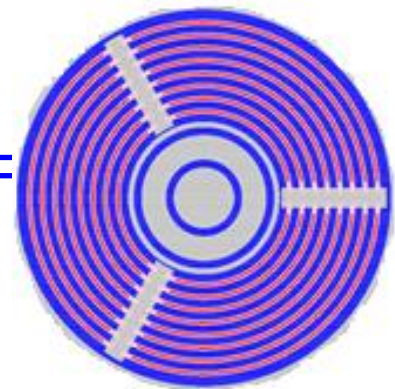
Fast neutron flux



Material ageing
(up to 16 dpa/y)

Applications :
Material and fuel samples irradiation/ageing
Radio-isotopes production for medical use
...

Geometry:
From 34 to 37 cylinder-shaped fuel assemblies
U3Si2-Al fuel enrichment of 19,75% then 27%
Aluminum racks (hosting all the fuel assemblies)
Hafnium control rods (in the center of fuel assemblies)
Beryllium reflector



Main Stakes

Radiations

Reduction of uncertainties in experimental conditions

- Neutrons / gamma flux
- Neutron spectrum
- Neutron fluence

Better knowledge of nuclear heating

→ design of experimental devices
Major stack for JHR

Physical parameters

Better characterization of thermo-hydraulic conditions

→ temperature field / fluid flows

Highly instrumented experiments

« Cook and look » → Online measurement of experimental parameters

Material behavior

New materials, better representativeness

- Temperature
- Deformations (creep...)

Fuel improvement and qualification

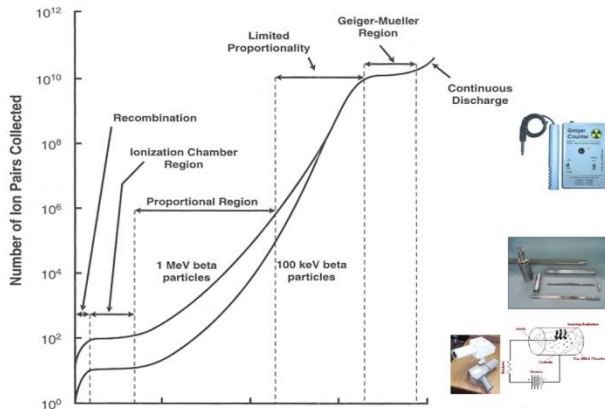
- Fission gas release
- Pellet-cladding interaction
- Accidental conditions (LOCA)

For nuclear reactor neutron detection and measurement

❖ On-line radiation detectors ❖

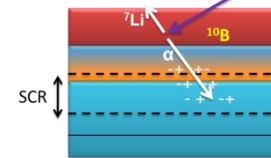
❑ Polarized Gaseous detectors

- Ionization chambers (FC, BLD)
- Proportional counters (CPNB, ^3He)
- Geiger-Müller counters



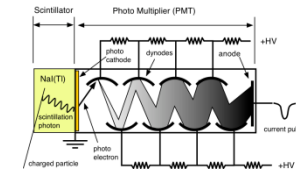
❑ Solid state radiation detectors

- Semiconductor detector (Si, Ge, SiC, CVD, GaAs, CdTe...)



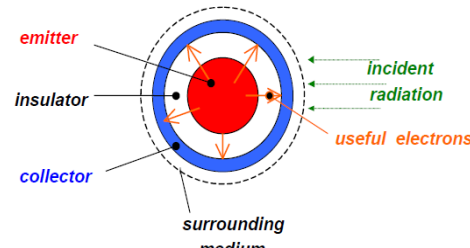
❑ Scintillator radiation detectors

- Cristal scintillator
- Organic scintillator
- Gaseous scintillator



❑ Non-polarized Gaseous detectors

- Self Powered Neutron Detectors (SPND)



Fission chambers

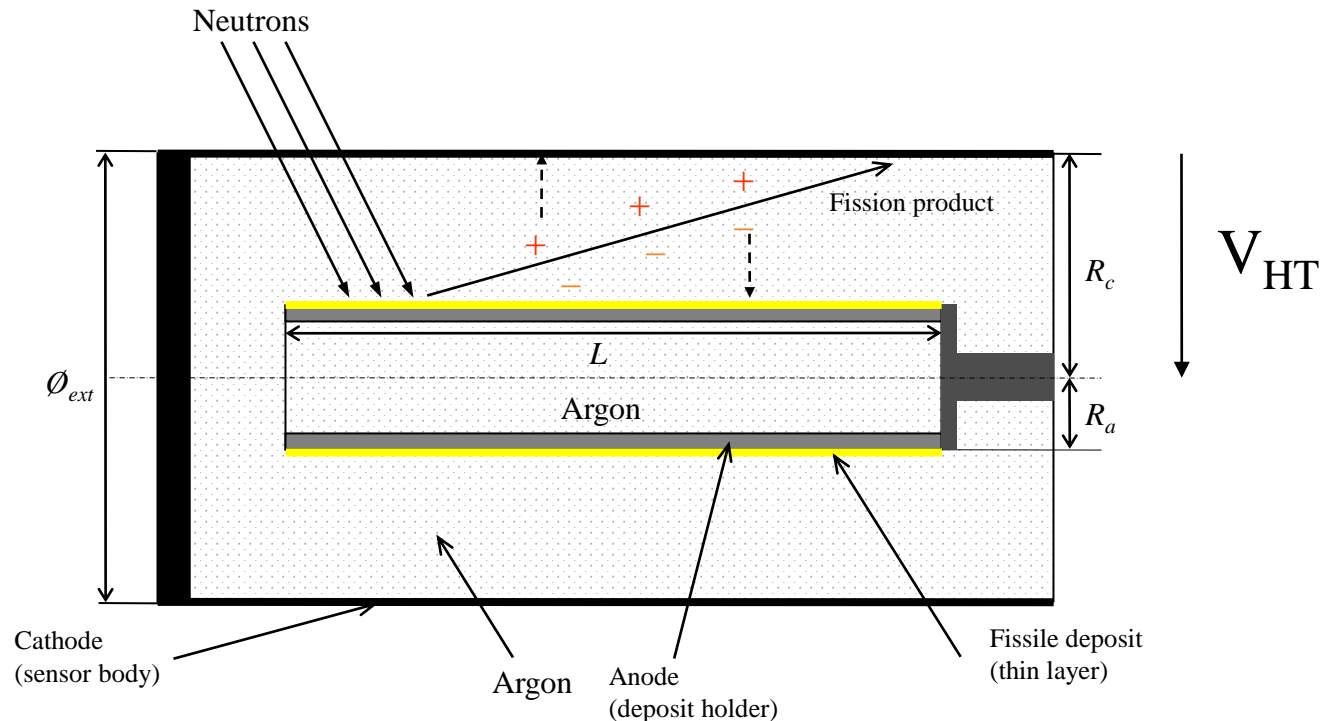
Online and real-time neutron flux measurement :

- ✓ Absolute/relative Measurements
- ✓ Amplitude of 10^6 to 10^{15} $n.cm^{-2}s^{-1}$
- ✓ Absolute or relative measurement (axial flux profile)

Principle: Measurement of the current generated by fission reactions in a fissile material (generally U_{235}) deposited on an electrode

$$s = f(\tau_f, V, P, \dots)$$

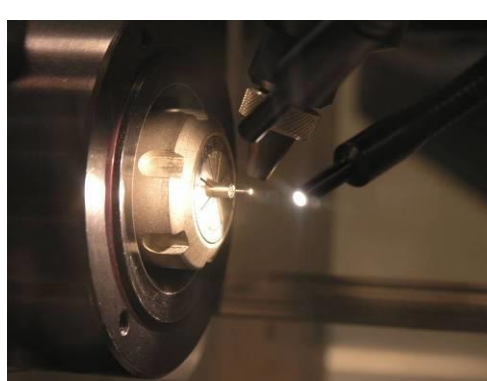
$$\tau_f = N \cdot \sigma \cdot \phi$$



CEA – Cadarache Fission Chamber Workshop

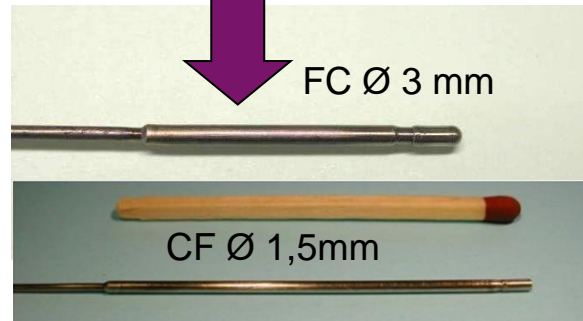
PHOTONIS

- Design and Realization of sub-miniature fission chambers
 - Characterized fissile deposits (mass/composition)
 - Optimized and dedicated Geometries
 - High temperature operation ($> 400^{\circ}\text{C}$)



Thorium 232
Uranium 233, 234, 235, 238
Neptunium 237
Plutonium 238, 239, 240, 241, 242
Américium 241
Curium 244

FC \varnothing 1,5mm / 4mm / 8 mm



FC \varnothing 3 mm

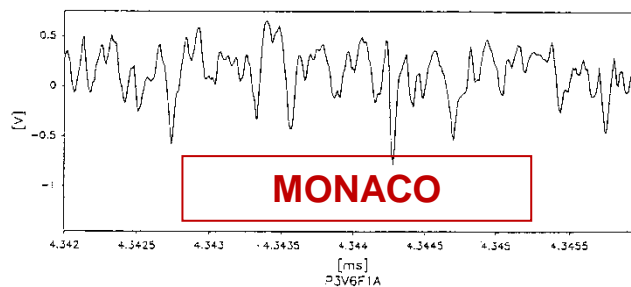
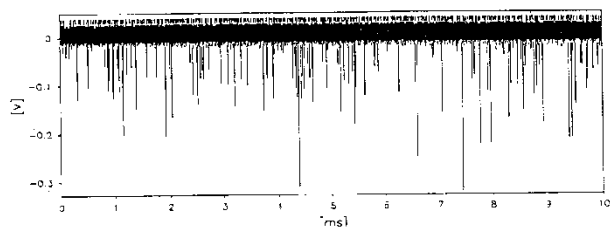
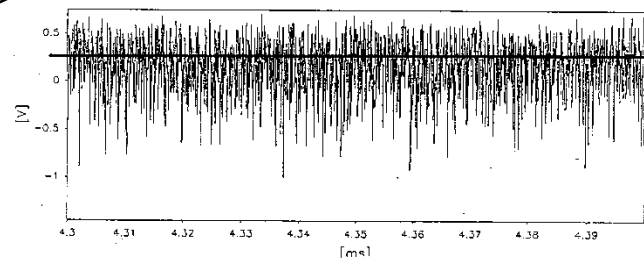
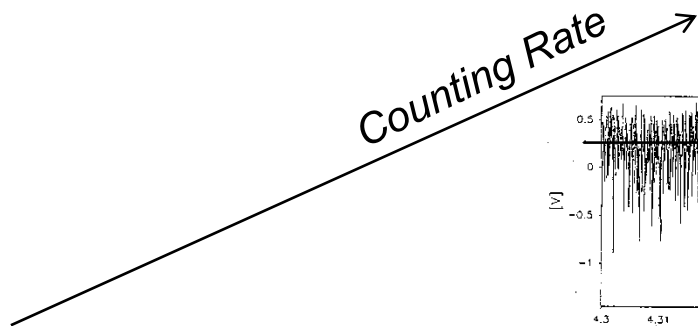
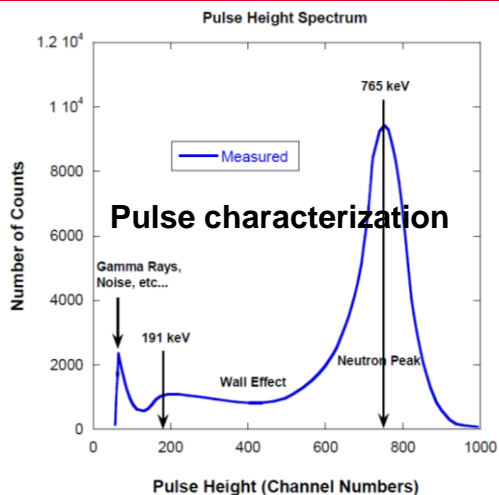
CF \varnothing 1,5mm

3-Bodies FC



On-line Neutron measurements

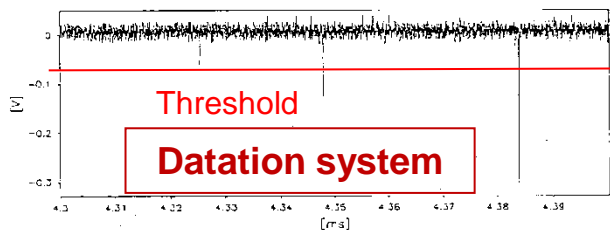
Fission Chamber operating modes



Current Mode

SPECTRON

MONACO



Fluctuation Mode (Campbell)

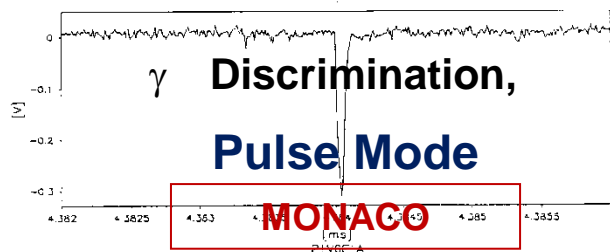
$$V^2 = KNQ^2$$

V^2 : variance

Q : Average charge per element ($Q_\gamma < Q_n$)

N : Average counting rate

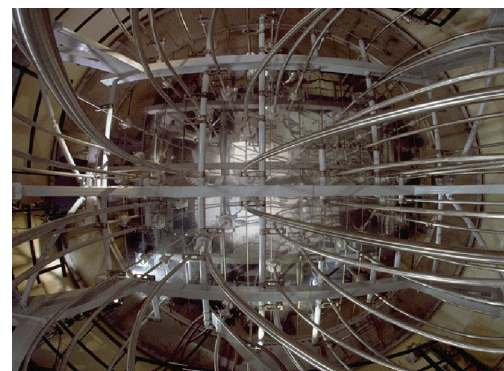
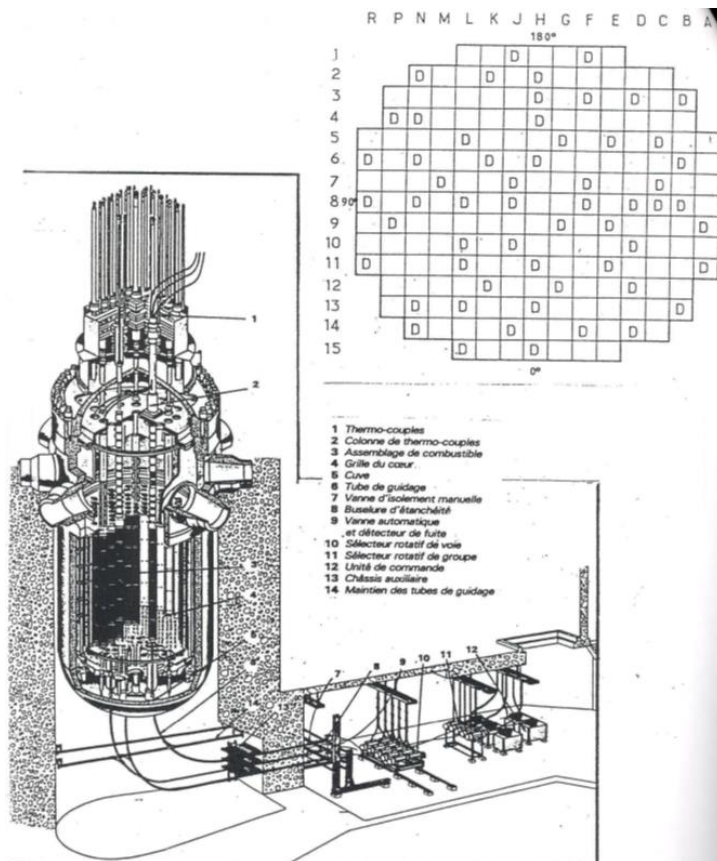
→ γ rejection



Mesure Incore

Pour les REP:
Chambre à fission mobile (CFUF43)
associée à une mesure du courant moyen via un câble minéral

Pour le réacteur RES (propulsion nucléaire):
Chambre à fission (CFUR64) associée à une mesure grande dynamique (modes impulsion, fluctuation et courant et via un câble minéral



Fond de cuve d'un réacteur de 1300 MWe

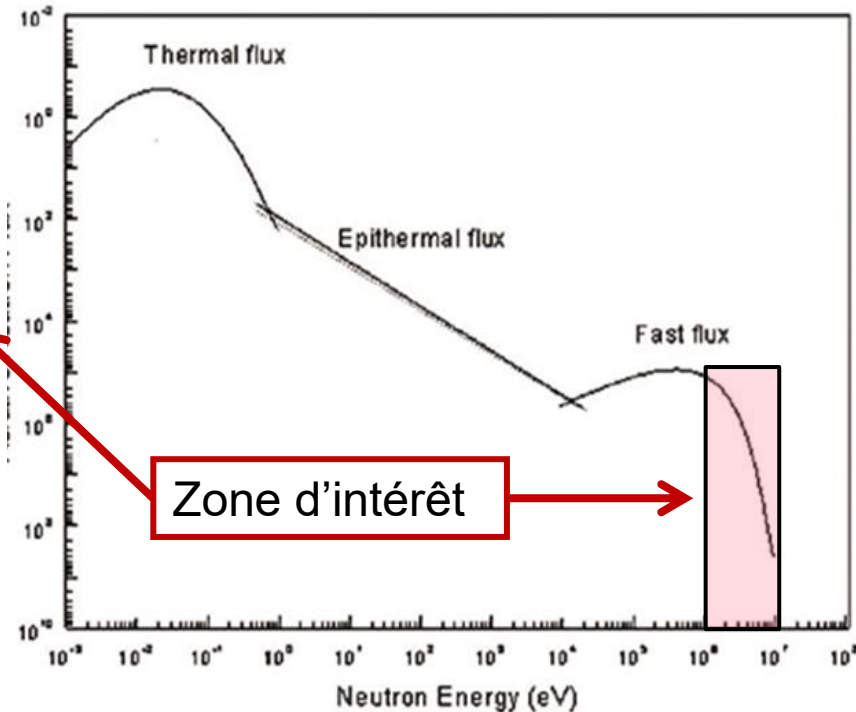
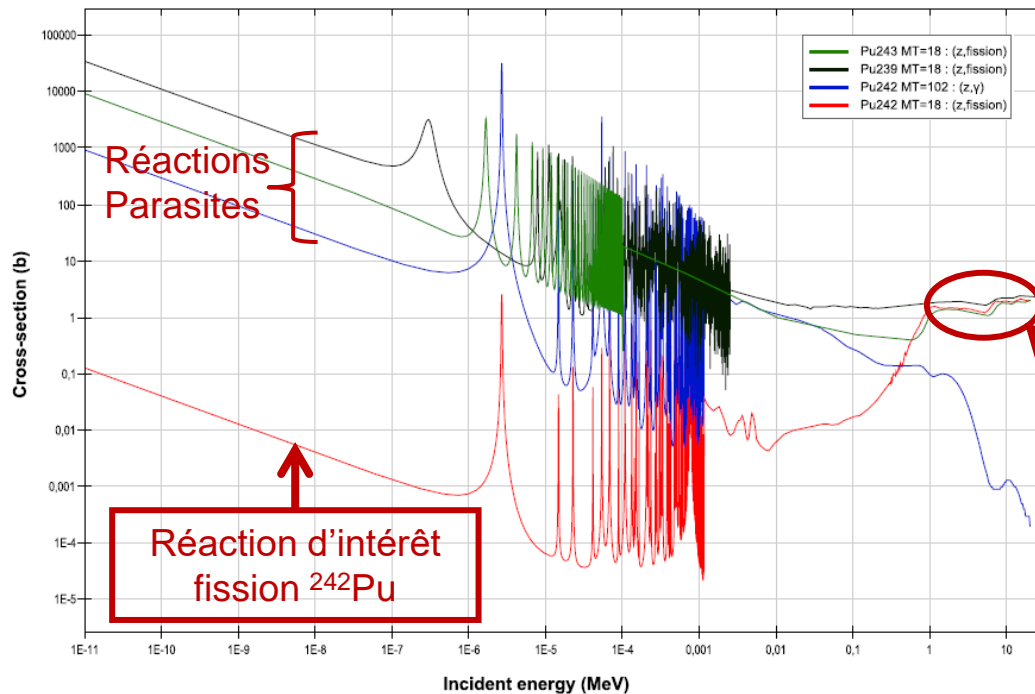
235U Fission Chamber : Predominant response to n_{th}

Fast Neutron Detector System - FNDS

Problématique

→ Measurement of neutron flux >1MeV is an indicator of Material Damage (dpa)

Incident neutron data / JEFF-3.2 /// Cross section



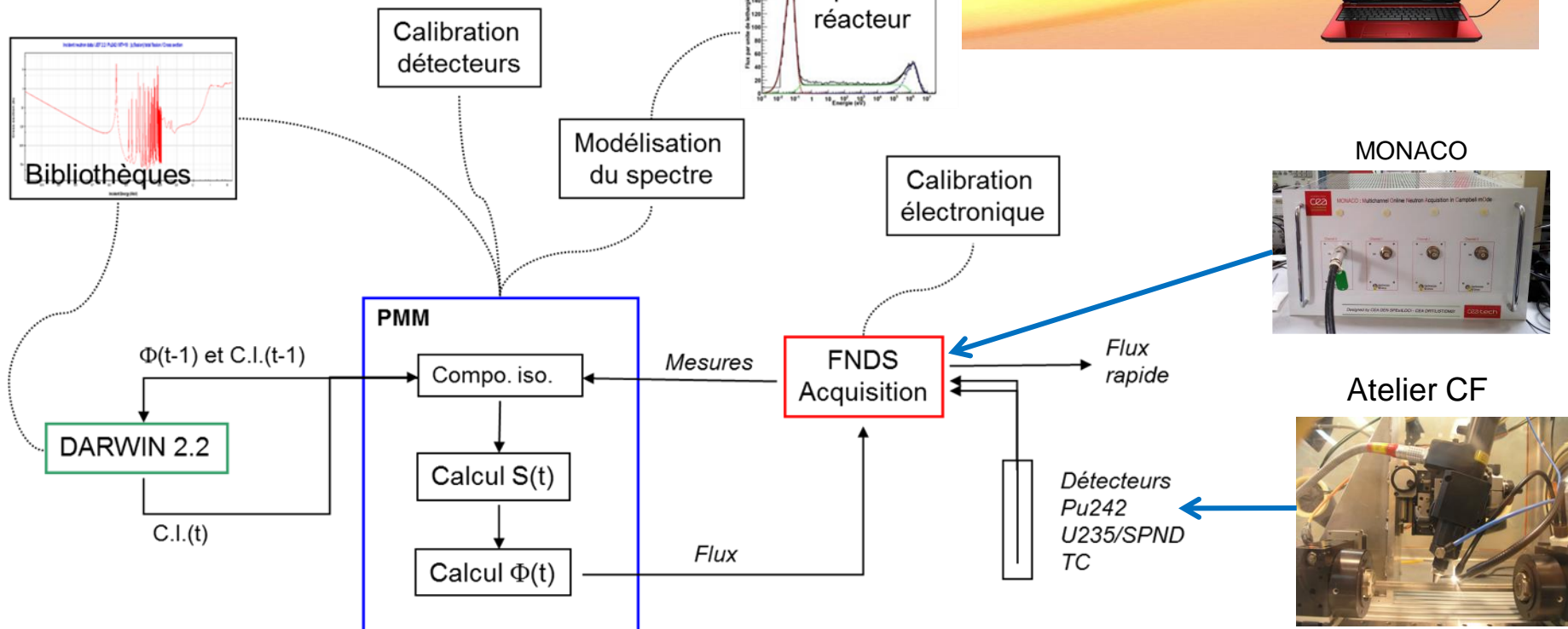
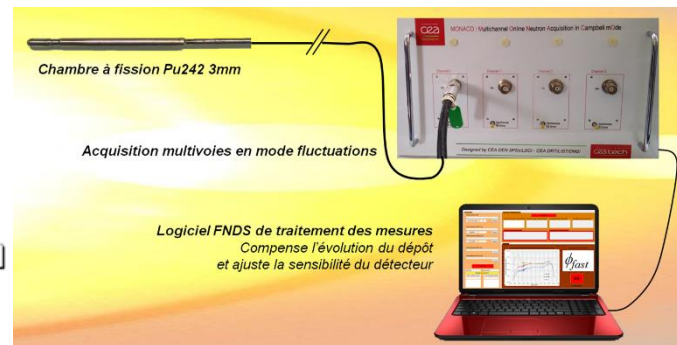
Chambre à fission pour neutrons rapides

Fast Neutron Detector System - FNDS



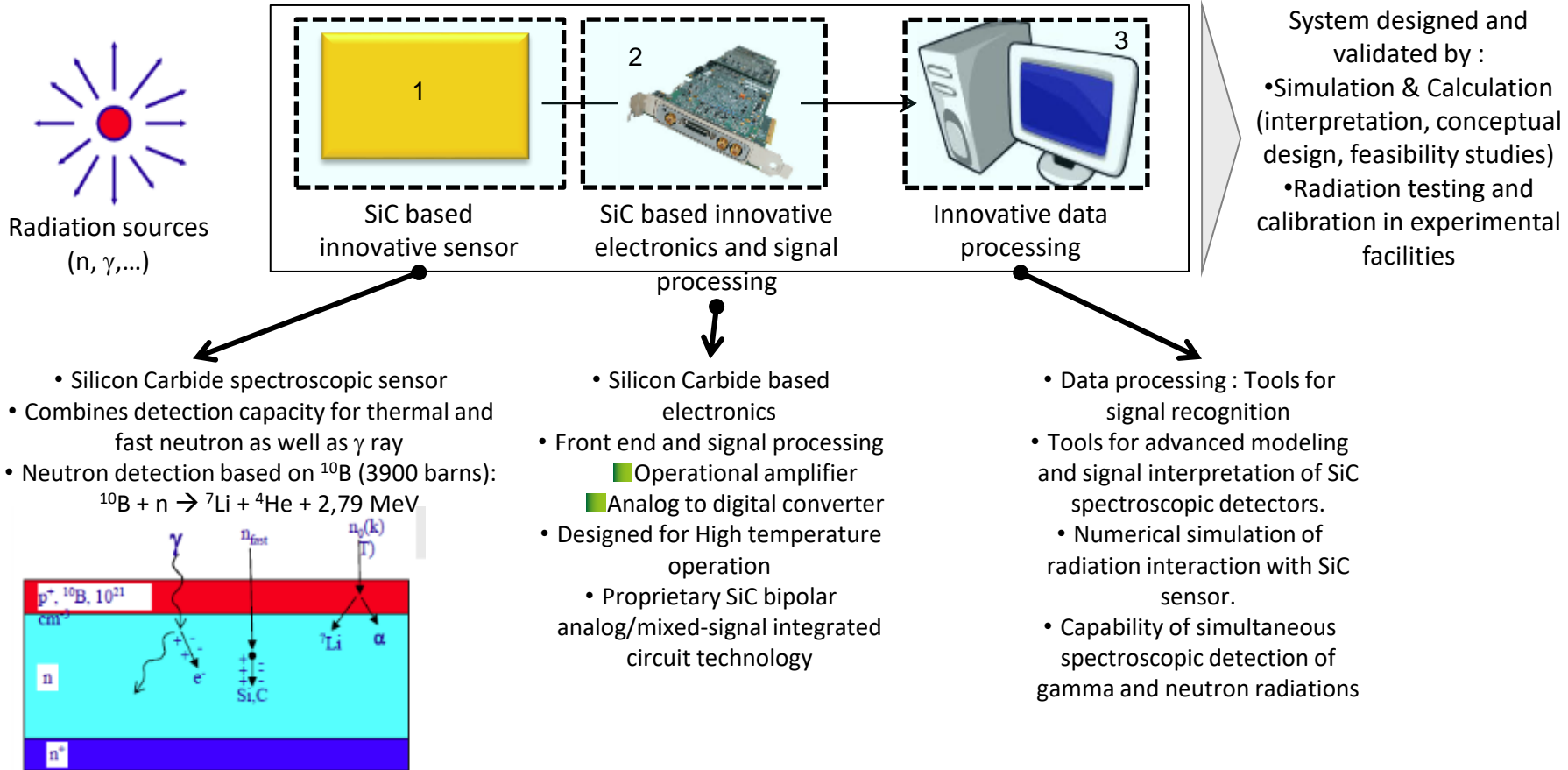
- On-line fast n-flux measurement ($E > 1 \text{ MeV}$, $\sigma < 10\%$)
- Flux max $10^{14} \text{ n/cm}^2/\text{s}$ - Fluence max 10^{20} n/cm^2

2 Patents



I-SMART : European Project aimed to develop and test advanced solide state sensors & measurement system for selective n-γ detection in Severe Media

I-SMART system : 3 sub-systems



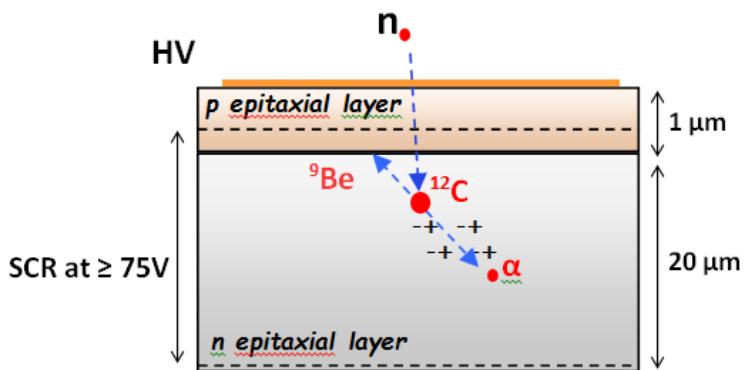
Why SiC and Diamond for Nuclear Detection ?

Property	Si	GaN	Diamond	4H-SiC
Bandgap (eV)	1.12	3.45	5.5	3.27
Break down field (MV cm ⁻¹)	0.3	2	10	3
e-hole creation energy (eV)	3.6	8.9	13	7.78
Threshold displacement energy (eV)	13-20	10-20	40-50	22-35
Thermal conductivity (W/cm·K)	1.5	1.3	22	4.9

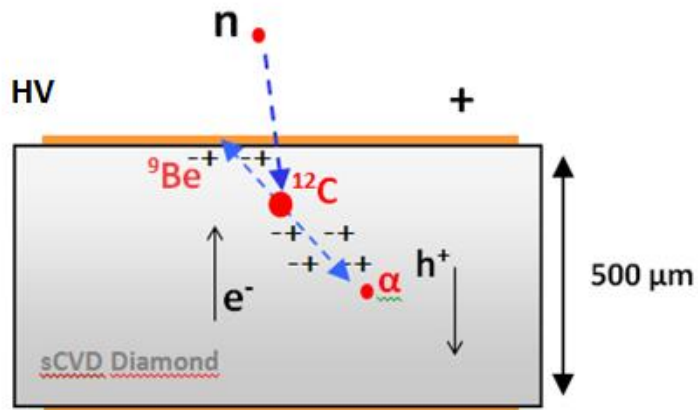
The main advantages of SiC and Diamond :

- Wide band gap : low leakage current
- High breakdown field : fast response (ns)
- High Energy threshold of defect formation : stability versus radiations
- High thermal conductivity : no cooling system required
- Carbon : good neutron/gamma discrimination
- Epitaxial Growth control (for SiC) : low defect concentration

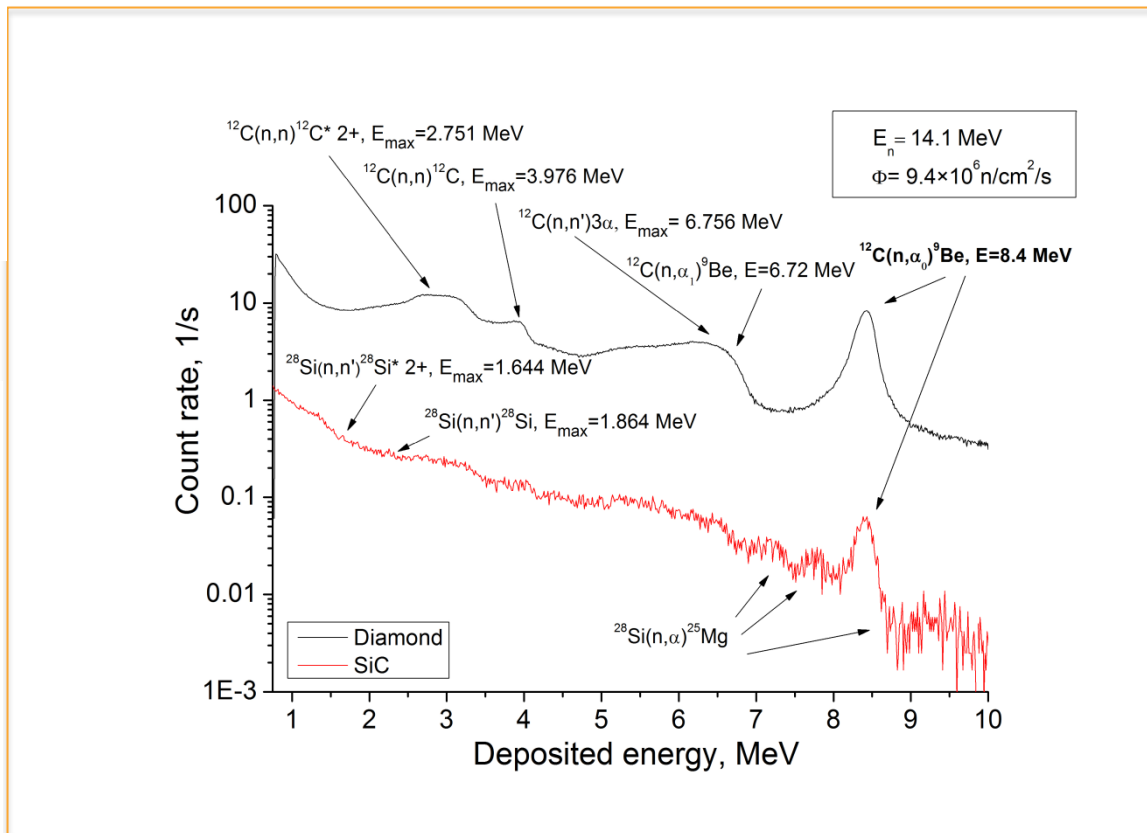
SiC detector (I_SMART project)



Diamond detector (Capacitor type configuration)



- D-T neutron generator
- $E_n = 14.1 \text{ MeV}$ (90°)
- $\Phi = 9.4 \times 10^6 \text{ cm}^{-2}\text{s}^{-1}$



SiC : lower intensity (thinner SCR), Si-related peaks

Thermal Neutron Detection / Tested @ Minerve ZPR reactor

GOALS

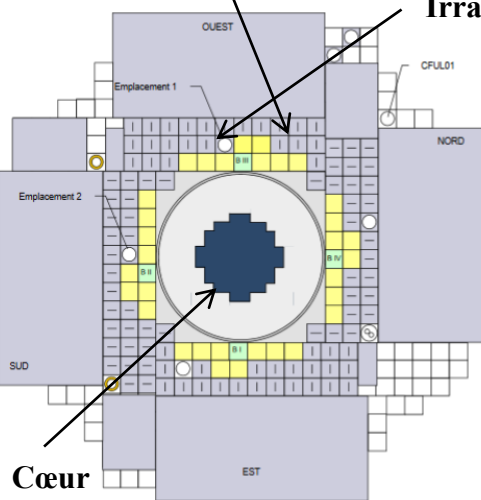
- CVD and SiC sensor response to thermal neutron flux in a mixed n/g field
- CVD and SiC stability and performance comparison

MINERVE

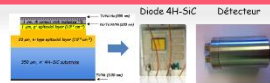
- ZPR – Nominal Power 100 W



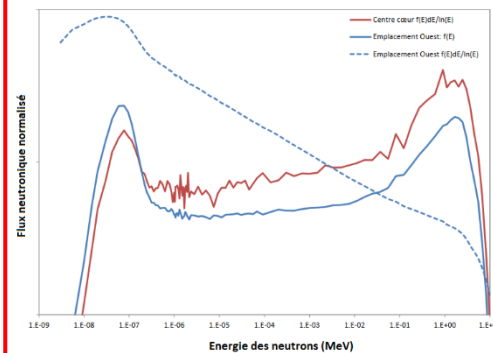
Reflector (graphite)



Irradiation Location $\Phi_{total} = 9.41 \times 10^8 \text{ n}/(\text{cm}^2 \cdot \text{s})$ à 80W

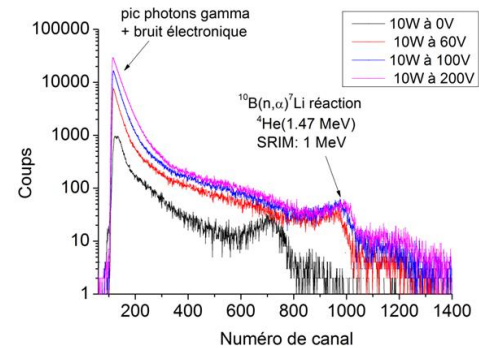


Calculated neutron flux (MCNP5 JEFF3.1.1)



Energie de neutrons	
$E_n < 0.5 \text{ eV}$	30.6%
$0.5 \text{ eV} < E_n < 100 \text{ keV}$	26.9%
$100 \text{ keV} < E_n < 1 \text{ MeV}$	26.5%
$1 \text{ MeV} < E_n < 5 \text{ MeV}$	14.8%
$5 \text{ MeV} < E_n < 10 \text{ MeV}$	1.2%

Also tested in BR1 @ $7.10^8 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (thermal flux).



Décalage du pic → Changement de la CCE

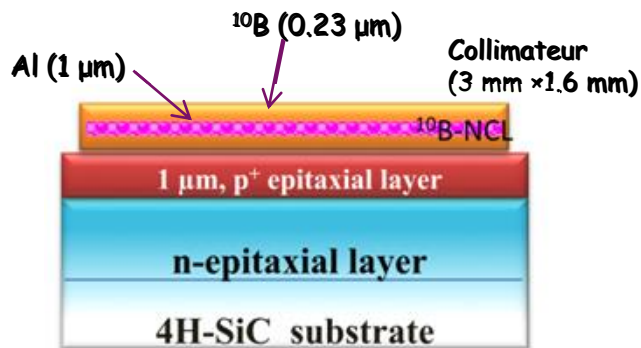
CCE incomplète à 0V
piégeage des porteurs de charge, recombinaison ?

Augmentation du comptage avec V dans le 1er pic

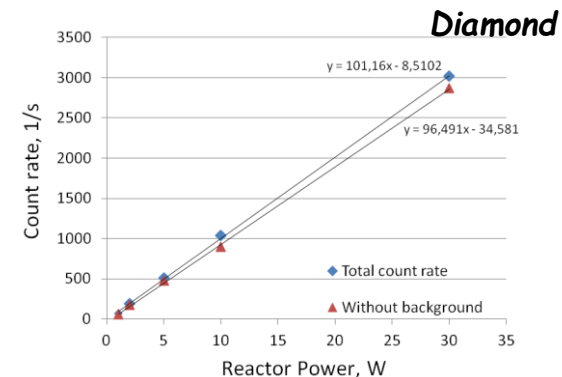
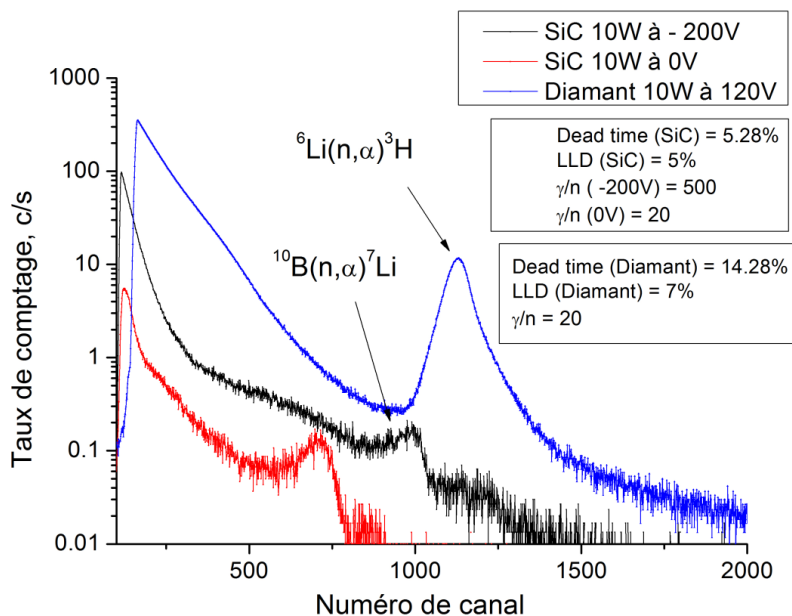
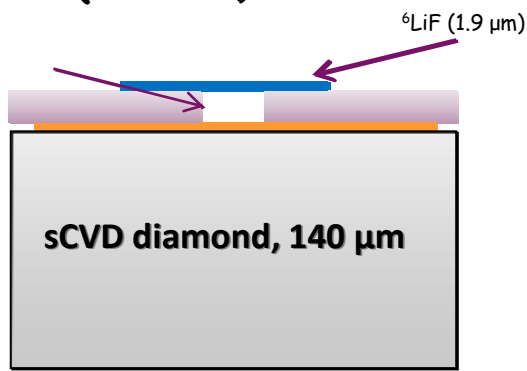
0 V : $W(ZCE) = 4.02 \mu\text{m}$
 -200 V : $W(ZCE) = W_n = 20 \mu\text{m}$
 Augmentation de la ZCE

Thermal Neutron Detection / Tested @ Minerve ZPR reactor

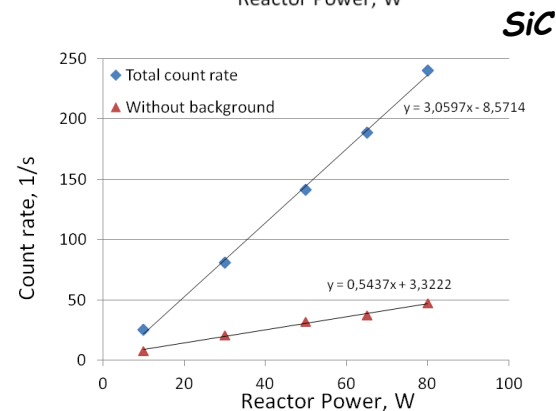
SiC detector (I_SMART project)



sCVD Diamond detector (CIVIDEC)



$\Phi_{total} = 9.41 \times 10^8 \text{ n/cm}^2\text{s}^{-1}$



SiC Detector :

- Better gamma/neutron discrimination at 0V (Full Detection)

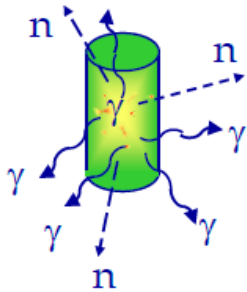
Diamond Detector :

- Higher count rate (higher C density)

- ❑ INTRODUCTION
- ❑ Specific features, constraints and requirements
- ❑ Examples of radiation measurements and associated instrumentation for :
 - In Pile Measurements
 - **Radioactive waste characterization and control**
- ❑ Conclusions and prospects

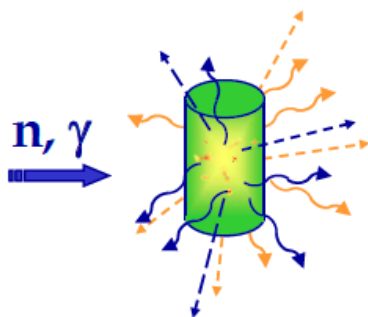
Radioactive Wastes Characterization & Management

Non Destructive Measurements (NDM)



- **Passive Measurements**

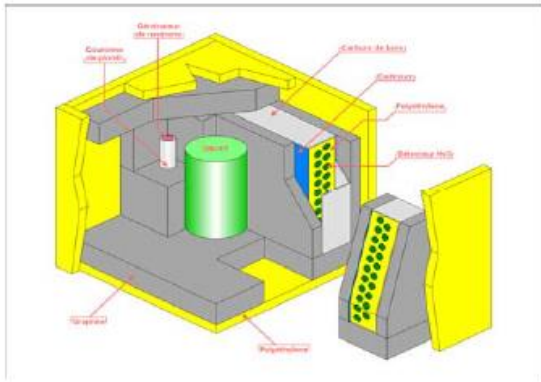
- photons : Dose rate, gamma spectrometry, gamma tomography
- neutrons : Global counting, neutron coincidences counting and neutron multiplicities counting.



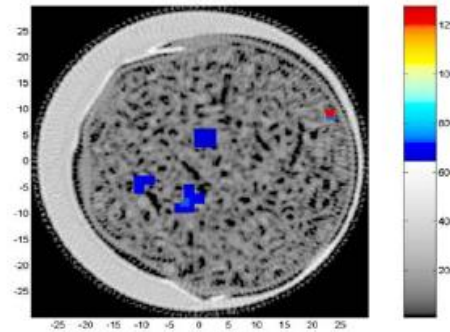
- **Actives measurement**

- Photon/Neutron Transmission Imagery/Radiography
- Neutron Interrogation \Rightarrow fission prompts and delayed neutrons, Gamma rays emission from $(n, n'\gamma)$, (n, γ) and following neutron activation reactions (n, p) , (n, α) ...
- Photon interrogation \Rightarrow delayed neutrons and gamma from photofission, Gamma rays from photon activation

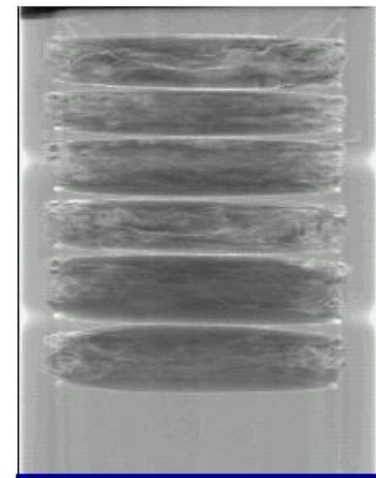
MEASUREMENT & INSTRUMENTATION FOR NDA



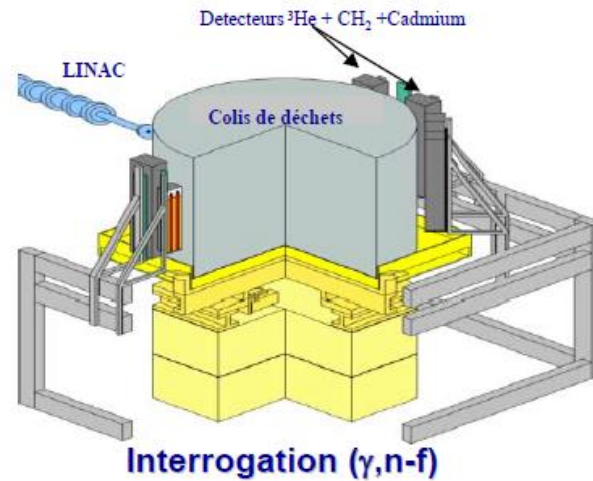
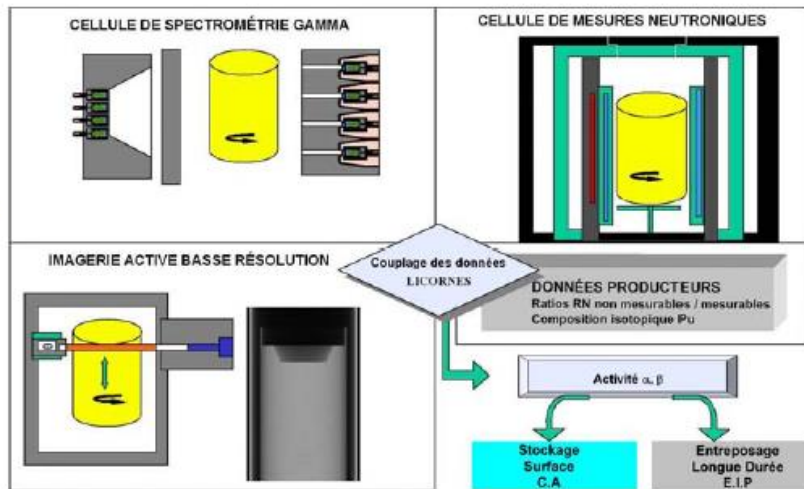
Interrogation (n,f)



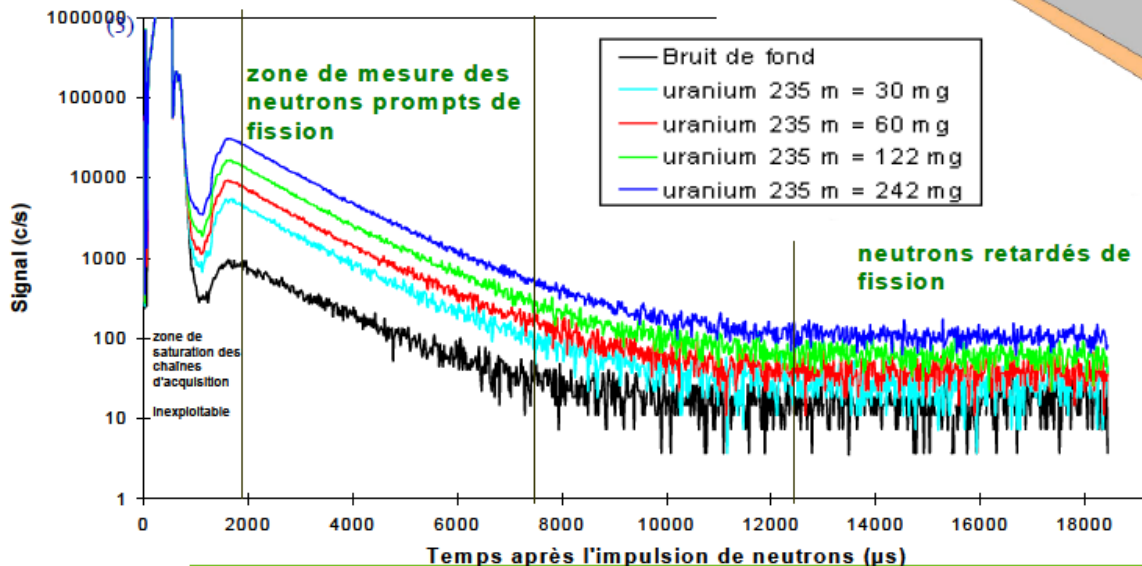
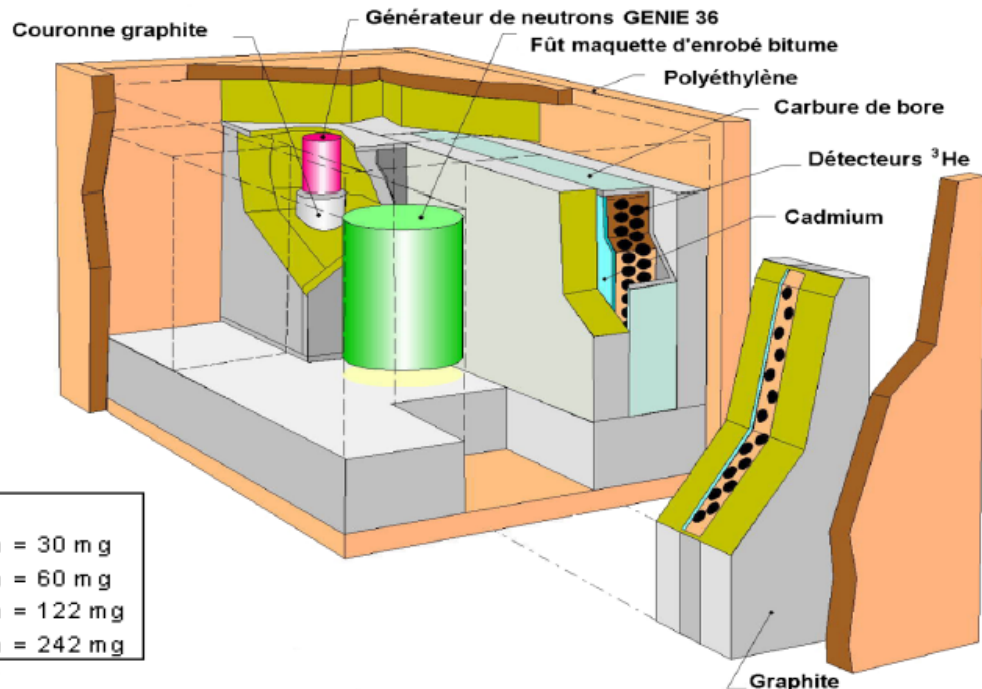
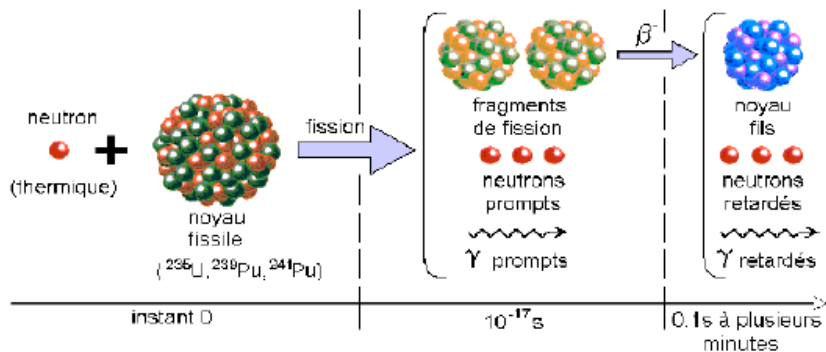
Photon Imagery and tomography



Combination



Active neutron measurement



- Neutron prompt signal

$$SP = a \times m(^{239}\text{Pu}) + b \times m(^{235}\text{U})$$

- Delayed neutron signal

$$\bullet SR = c \times m(^{239}\text{Pu}) + d \times m(^{235}\text{U}) + e \times m(^{17}\text{O}) + f \times m(^{238}\text{U}) + f \times m(^{232}\text{Th})$$

MEASUREMENT & INSTRUMENTATION FOR NDA

Active Photon Interrogation

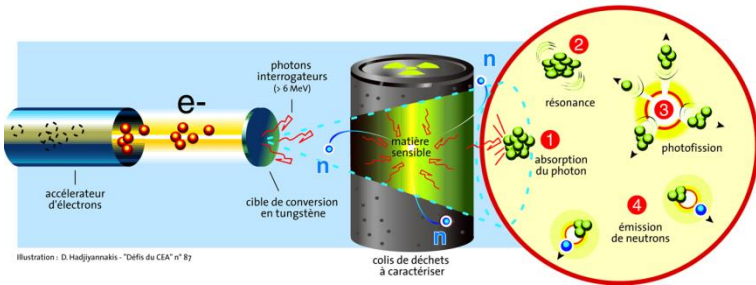
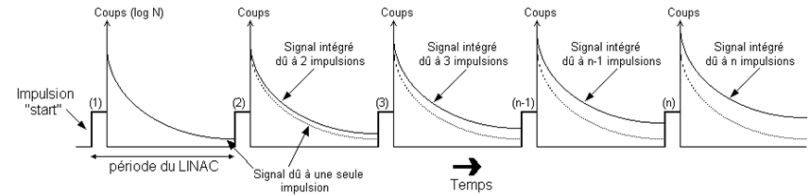
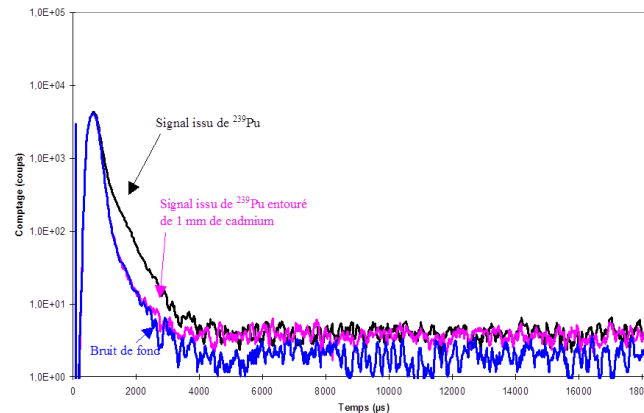
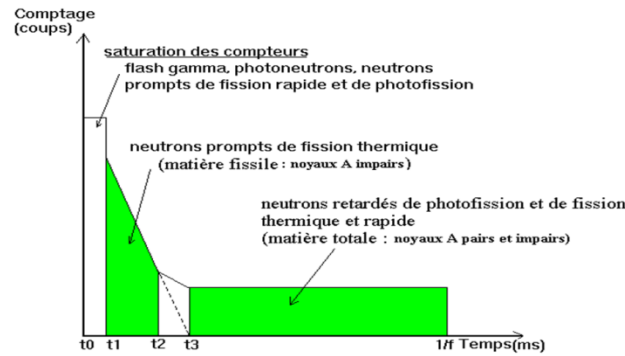
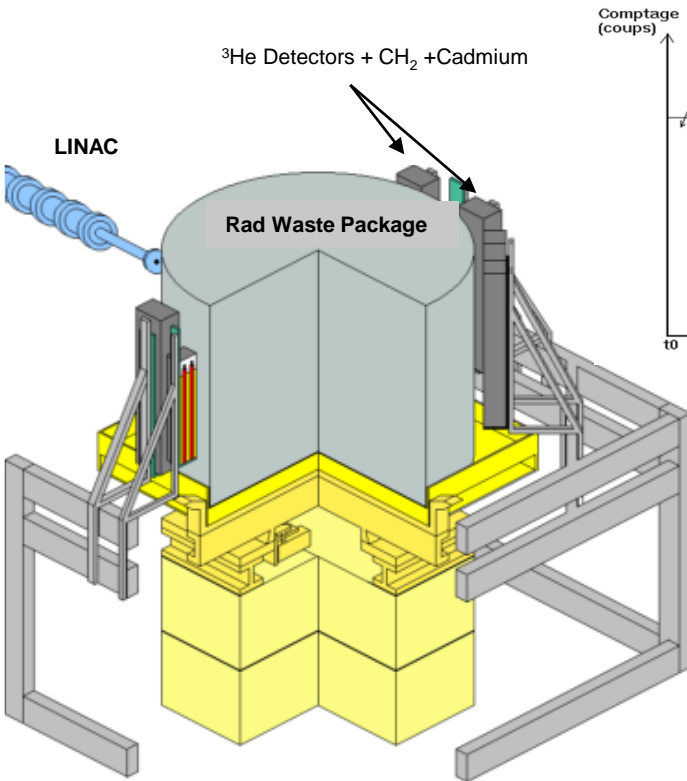
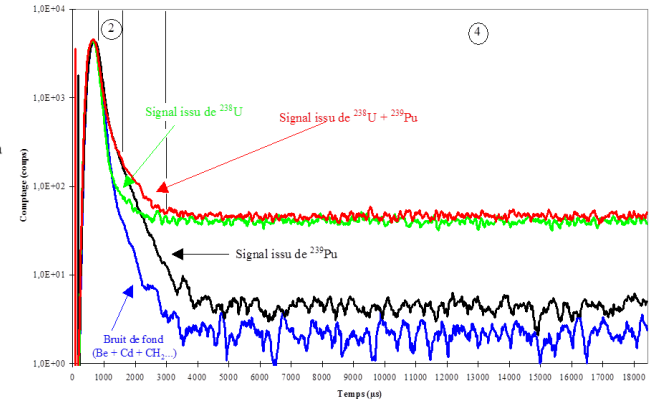


Illustration : D. Hadjipantakis - "Défis du CEA" n° 87

SPHINCS counting technique




Time distribution of neutron signal for simultaneous photon & neutron interrogation (Ee = 15 MeV, Cu & Be)



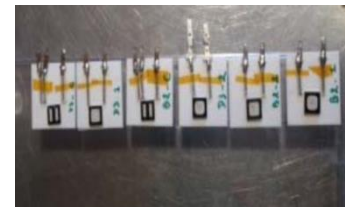
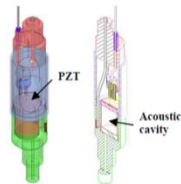
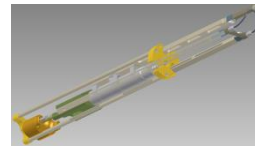
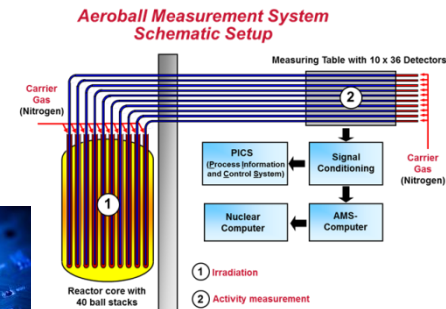
Prompt neutron signal comes from thermal neutron interrogation and delayed neutron signal is mainly due to photofission interrogation (~95%)

- **Important R&D efforts** are maintained on instrumentation and measurement dedicated to existing and future Research Reactors (JHR,...)
- As codes and nuclear data get more and more accurate, **nuclear instrumentation** should be **continuously improved** in terms of:
 - **Uncertainties and Precision** → Absolute measurements
 - **Reliability** to support high fluence (up to 100dpa !) and temperatures
 - **Measuring and Interpretation Processes** (online / combined measurements)
- Due to the closing of several **irradiation facilities** and the disappearance of the associated teams, **collaborations** are a favoured way for instrumentation developments to be enhanced →

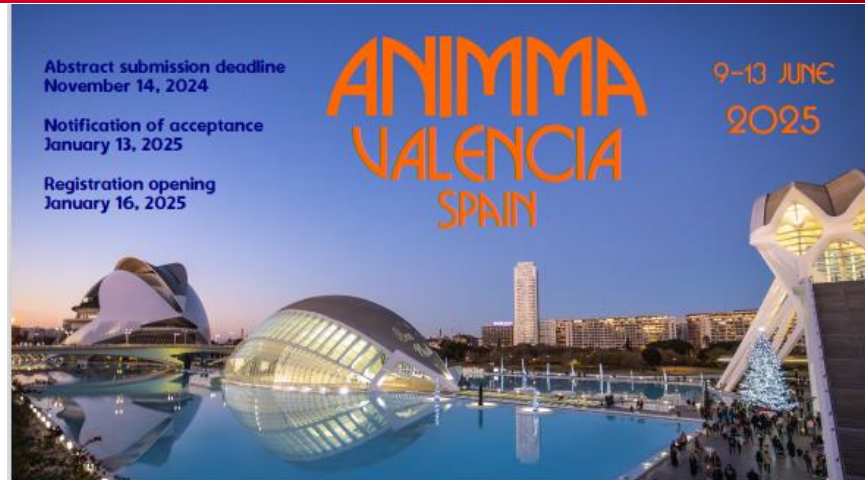

- An attention should be paid for **Nuclear Data** which are often not enough developed for instrumentation needs (e. g. charged particles)

Maintain and enhance efforts on Research and Innovation in :

- High temperature measurements (500°C up to 1000°C).
- High radiation level measurements
- High count rate measurements
- Selective radiation measurements n , γ
- Neutron spectrum measurements
- Material and electronics hardening
- Integrated electronics
- Multiplexing
- Integration probes
- Accurate modeling/calculation tools (nuclear data library “corrections”)
- Real time data acquisition
- Combined measurements and cross interpretation and analy**
- Uncertainties treatment, analysis and reduction
- Data mining, Algorithmic, Machine learning, Artificial Intelligence
- Numerical Twins
-



ANIMMA



The 9th International Conference on Advancements in Nuclear Instrumentation Measurement Methods and their Applications

APPLICATION FIELDS

- Fundamental Physics
- Space Sciences and Technology
- Fusion Diagnostics and Technology
- Research Reactors and Particle Accelerators
- Nuclear Power Reactors and Nuclear Fuel Cycle
- Nuclear Safeguards, Homeland Security and CBRN
- Decommissioning, Dismantling and Remote Handling
- Severe Accident Monitoring
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- Current Trends in Development of Radiation Detectors
- Education, Training and Outreach



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THANK YOU

“It doesn’t matter how beautiful your theory is, it doesn’t matter how smart you are, if doesn’t agree with experiment, it is wrong.”
Richard Feynman (1918-1988)

