

# A Prototype Compact Accelerator-based Neutron Source for Canada for Medical and Scientific Applications

**Dalini D. Maharaj**

2022 CAP Congress – Monday 6<sup>th</sup> June 2022



University  
of Windsor

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## **Overview**

- Why does Canada need neutrons?
- Overview of Compact Accelerator-based Neutron Sources (CANS)
- The Prototype Canadian CANS
- Objectives of Target Moderator Reflector Optimization
- Current Studies and Outlook



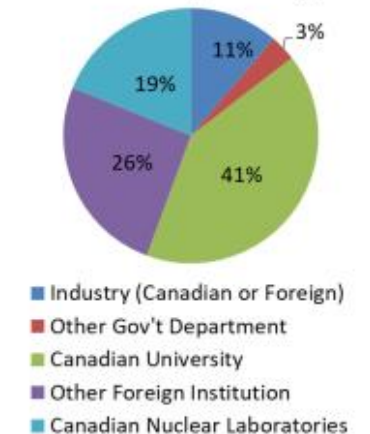
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# Current Status of Neutron Beams in Canada

- **Neutron Gap** - National Research Universal (NRU) reactor shut down in 2018
- McMaster Nuclear Reactor - only source of neutron beams in Canada

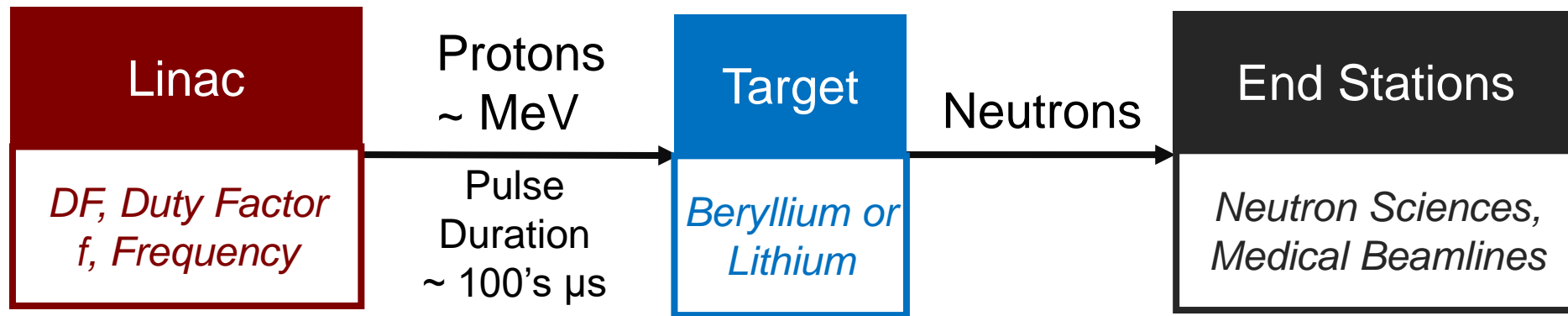


Beam Time by User Type



- Similar story globally - major research reactors closed e.g. BER-II, JEEP and Orphée
- Need new (affordable) pathways for neutron production
- Demand for high brilliance, pulsed neutron beams in Canada

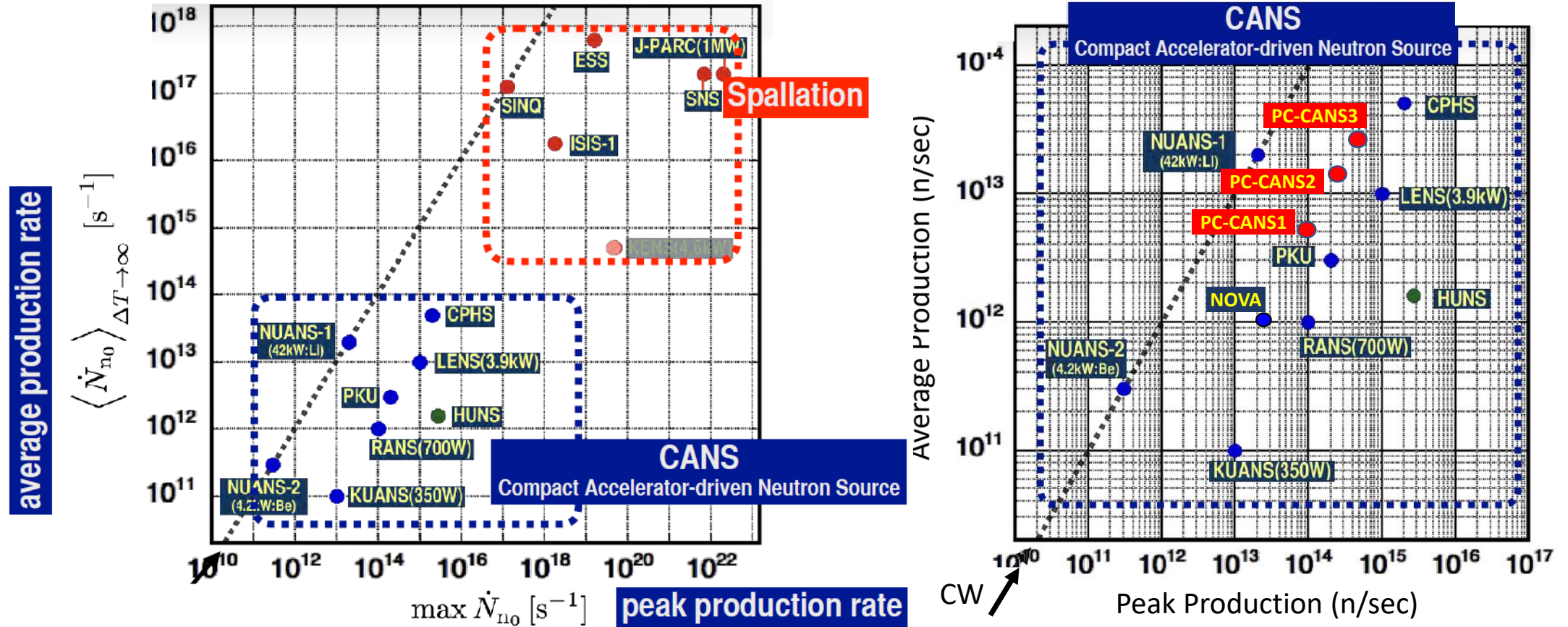
# The Compact Accelerator-based Neutron Source (CANS) Concept



## Advantages

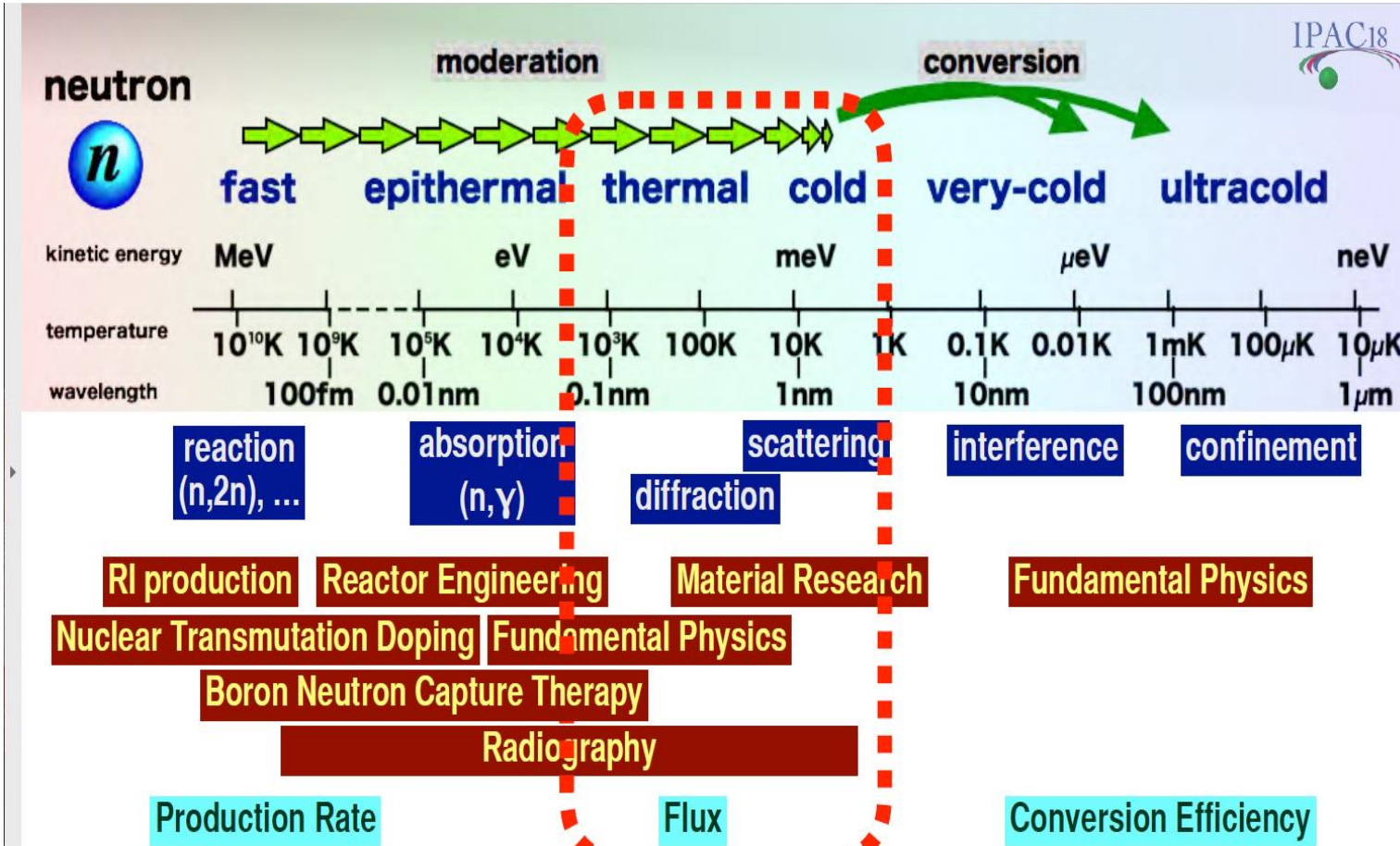
- I. **Compact** - less shielding required
- II. **Lower cost** when compared with reactor and spallation sources
- III. **High brilliance, pulsed neutron beams** realized
- IV. **Scalable** technology via
  - Boosting proton energy
  - Increasing accelerator current

# Global Neutron Landscape



- CANS provide neutrons to serve **most** user needs
- PC CANS designed to be competitive against similar scale sources

# Neutron Beam Applications



## Neutron Sciences

### I. Thermal Neutrons

$10 \text{ meV} < E < 100 \text{ meV}$

e.g. diffraction to resolve crystal structures  $\sim$  Angstroms

### II. Cold Neutrons

$E < 10 \text{ meV}$

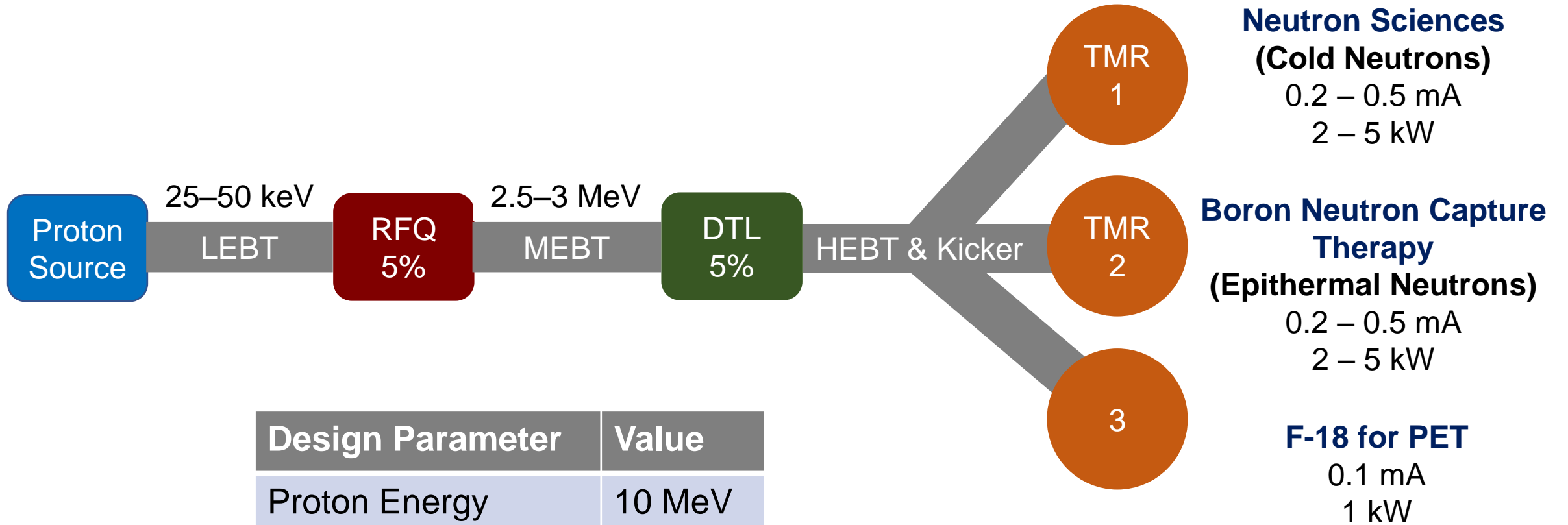
e.g. large scale structures

Boron Neutron Capture Therapy (BNCT)

Epithermal Neutrons

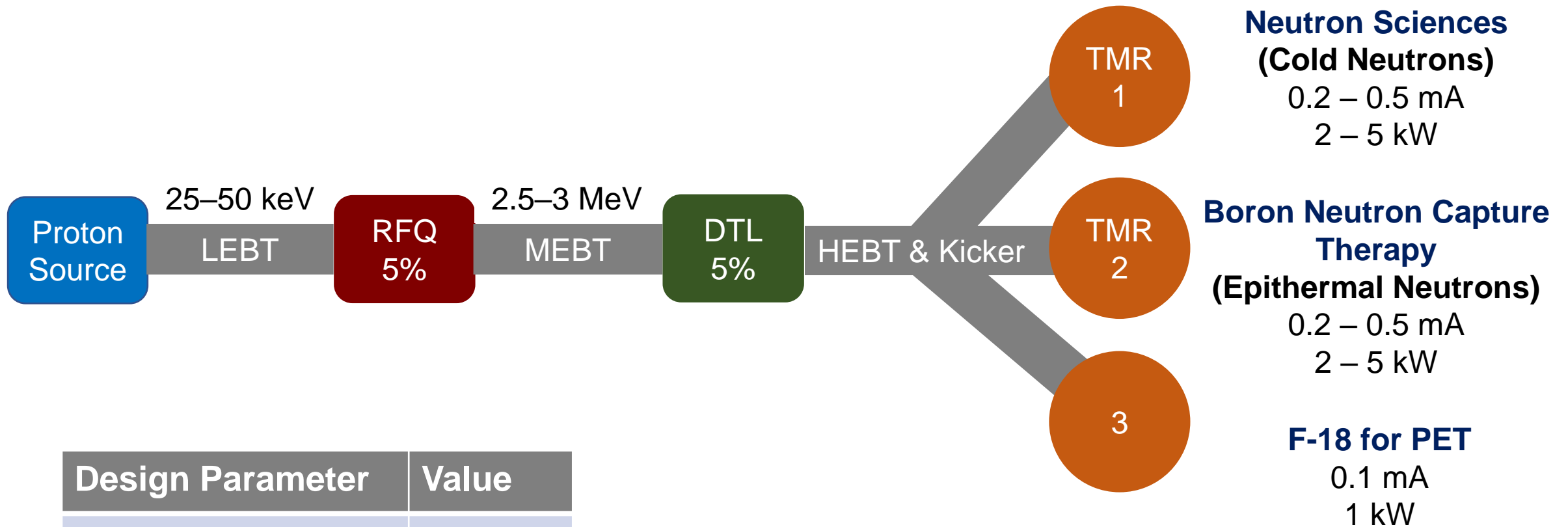
$0.5 \text{ eV} < E < 10 \text{ keV}$

# Overview and Objectives of the PC CANS



Design Parameter	Value
Proton Energy	10 MeV
Duty Cycle	5%
Total Peak Current	20 mA

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• Details of linac conceptual designs - [see Mina Abbaslou's talk Today at 4:30pm in the DAPI session, M3-6](#)



# Performance at End Stations

	<b>NOVA</b>	<b>PC-CANS 1</b>	<b>PC-CANS 2</b>	<b>PC-CANS 3</b>
Neutron yield average	$9.6 \times 10^{11}$	$4.8 \times 10^{12}$	$1.2 \times 10^{13}$	$2.4 \times 10^{13}$
Neutron yield peak	$2.4 \times 10^{13}$	$9.6 \times 10^{13}$	$2.4 \times 10^{14}$	$4.8 \times 10^{14}$
SANS yield n/s	$7.0 \times 10^4$	$3.5 \times 10^5$	$9 \times 10^5$	$1.8 \times 10^6$
BNCT n/s average		$1 \times 10^8$	$2.5 \times 10^8$	$5 \times 10^8$
$^{18}\text{F}$ saturated yield (GBq)		240		

**I. Small-angle neutron scattering** - High brilliance, pulsed, cold neutron beams of duration, 0.1-0.8 ms, at repetition rates of  $\approx 50$  Hz

**II. Boron Neutron Capture Therapy** - Therapeutic epithermal neutron flux of  $> 1 \times 10^8$  n/s are possible, enabling a BNCT R&D station

**III. F-18 Isotope Production for PET** - Competitive rates for F-18 production

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} TMR  
} Optimization  
} Important

# Baseline Target Moderator Reflector (TMR) System for Neutron Sciences

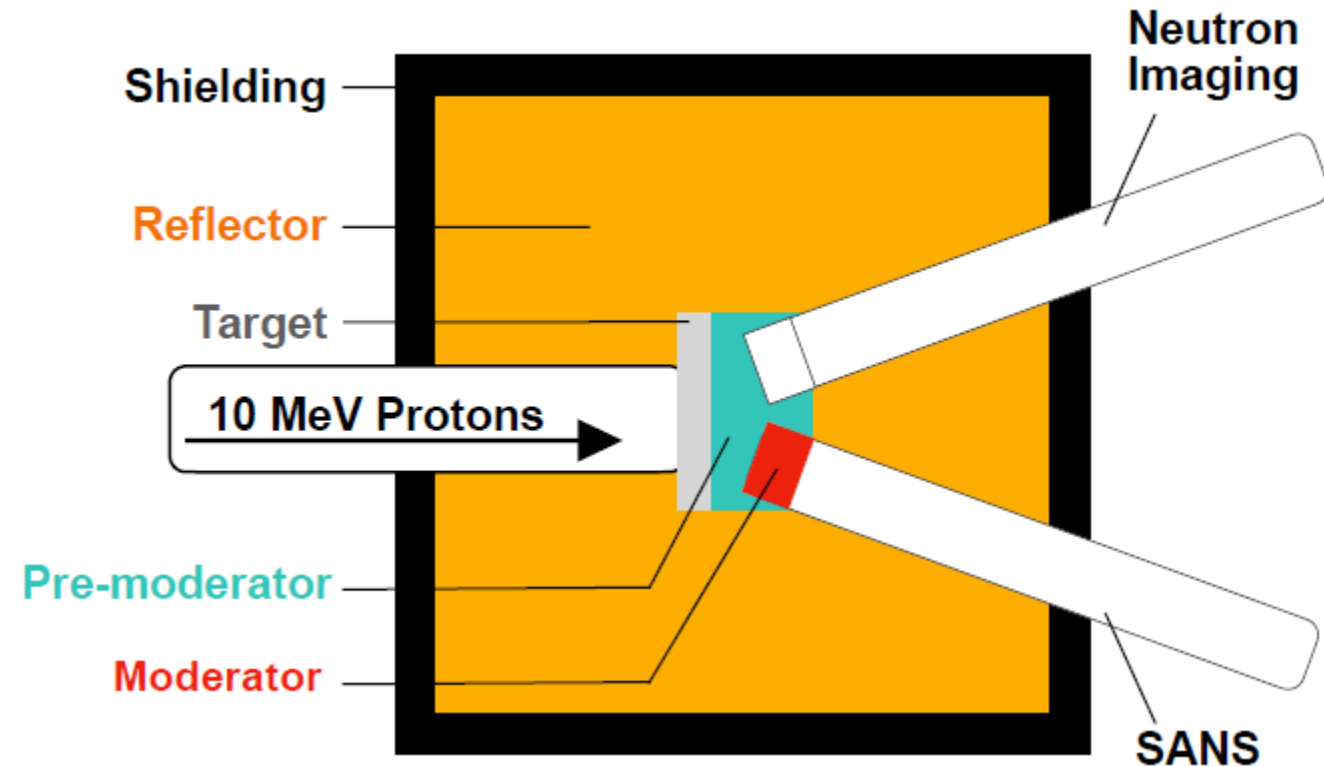
**Beryllium Target** – Produces neutrons via stripping reactions

**Pre-moderator** – slows neutrons from  $\sim$  MeV to thermal energies  $\sim$  10-100 meV

**Moderator** – slows thermal neutrons to cold energies  $<$  10 meV

**Reflector** – backscatters high energy neutrons for further moderation

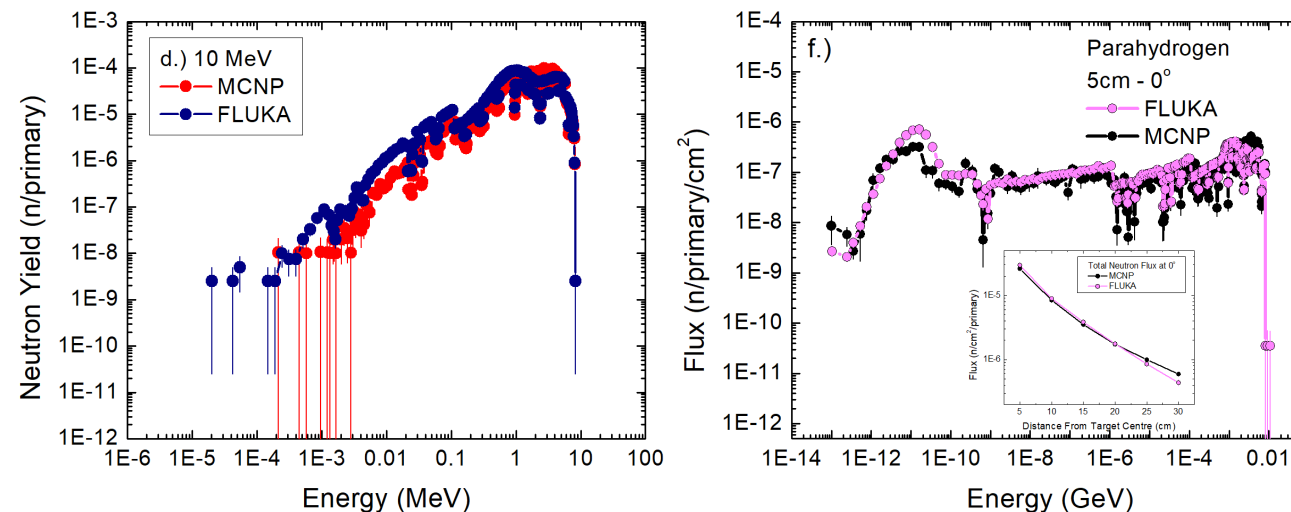
**Shielding** – protects users from exposure to harmful radiation



Tools Utilized  
FLUKA & MCNP

# Simulation Tools for Target Moderator Optimization

- FLUKA optimized for high energy particle transport but agrees well at 10 MeV
- Custom cross sections for moderator materials in MCNP
- Target-Moderator-Reflector studies for cold neutron beamlines in MCNP
- Target development and shielding studies in FLUKA

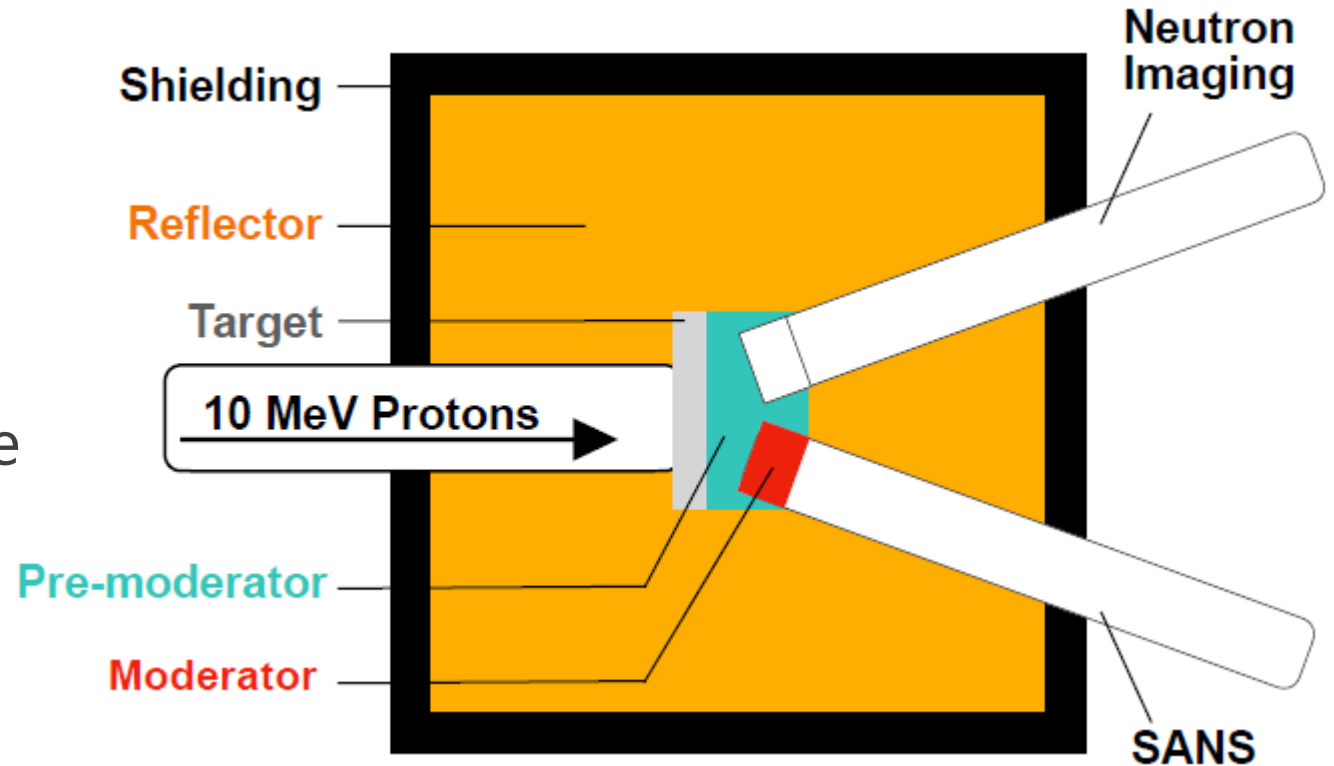


[\[1\] R. Laxdal, Journal of Neutron Research \*\*23\*\* 99-117, \(2021\).](#)

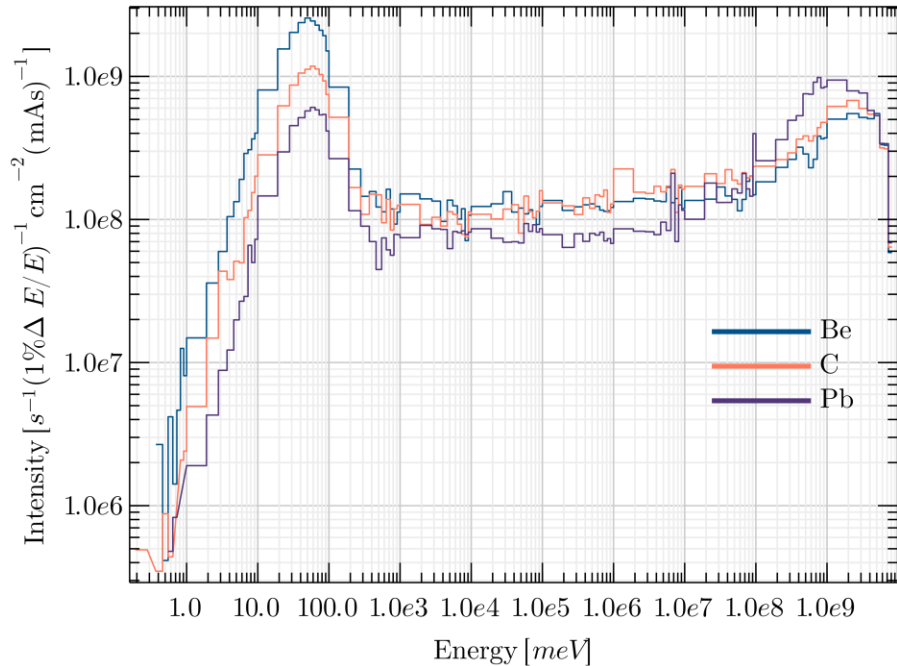
[\[2\] D. D. Maharaj \*et al\*, arXiv:2205.01662v1 \[physics.acc-ph\] \(2022\).](#)

# Objectives of Target Moderator Reflector Design & Optimization

- **Optimize neutron yields**
- **Optimize neutron time structure and spectra**
  - (i) Each neutron instrument has its own requirements for pulse structure
  - (ii) Influence of proton time structure on neutron time structure
  - (ii) Materials selected for TMR affect neutron time structure and neutron spectrum delivered



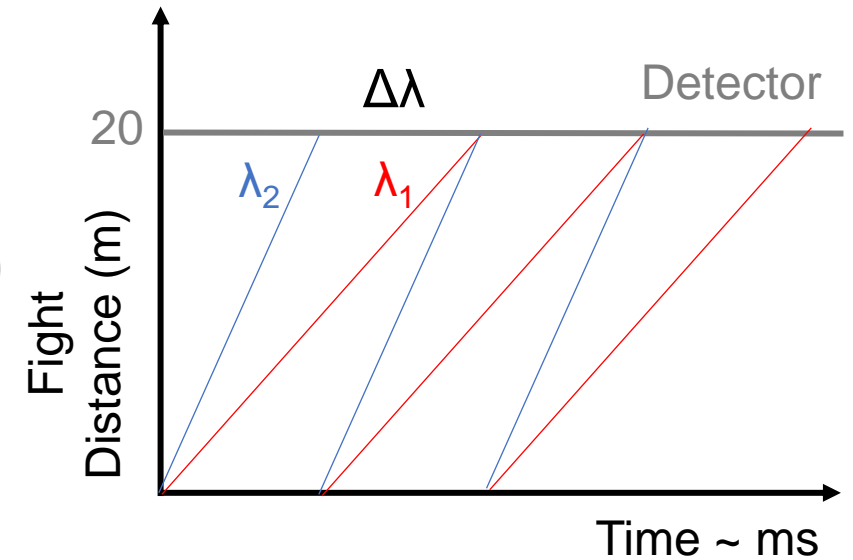
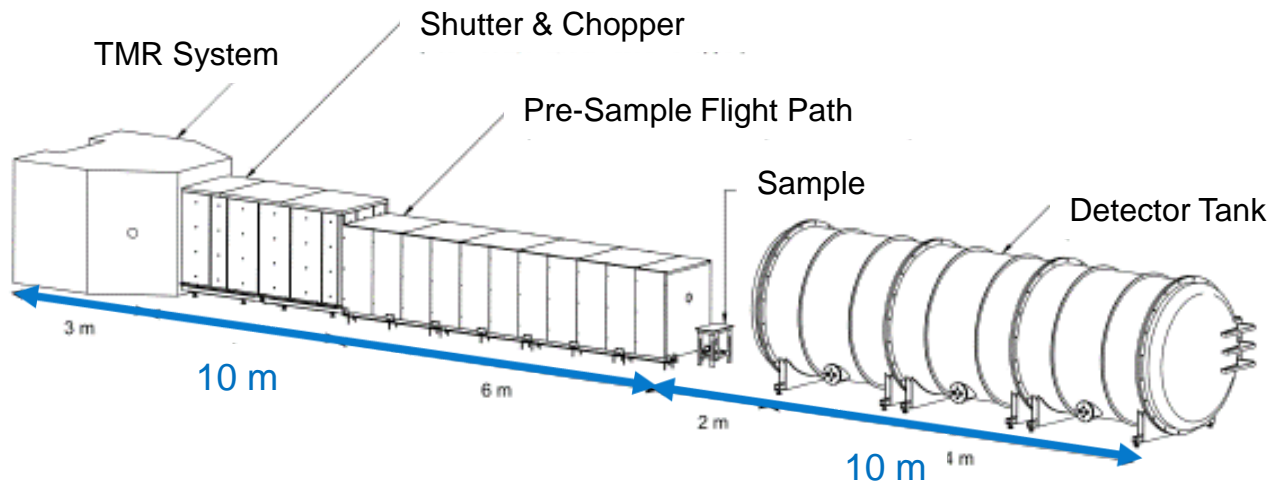
# Influence of Reflector Selection on Neutron Yield



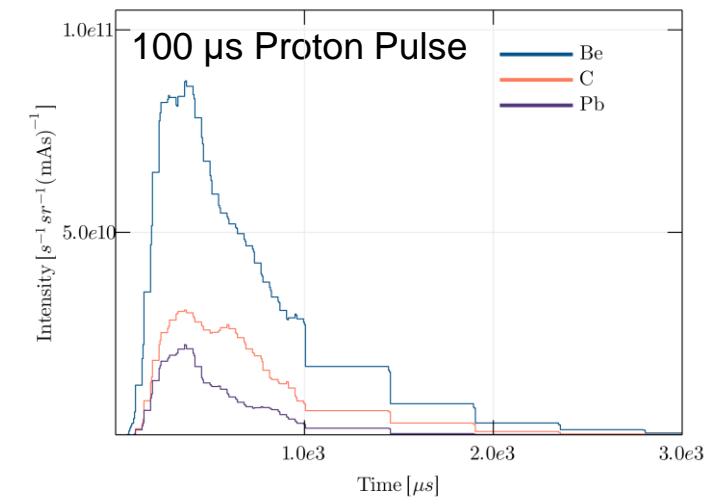
Neutron flux for Mesitylene cold moderator ( $n/cm^2/mC/s$ )			
	Lead	Graphite	Beryllium
Cold Flux	$3.92 \times 10^7$	$1.09 \times 10^8$	$2.68 \times 10^8$
Thermal Flux	$8.07 \times 10^8$	$1.70 \times 10^9$	$3.88 \times 10^9$
Total Flux	$4.39 \times 10^9$	$5.82 \times 10^9$	$7.99 \times 10^9$

- Performance  $Be > C > Pb$  with respect to neutron yield
- Beryllium and graphite,
  - Fast neutron spectrum is significantly suppressed
  - Thermal neutron yields are higher

# Matching Proton Pulse Structures with SANS Requirements



- 20 m SANS instrument delivers cold neutron bandwidth,  $\lambda_1 < \Delta\lambda < \lambda_2$
- Proton pulse duration, source frequency and duty factor, chosen to ensure neutron pulses (or frames) are well separated when they arrive at the detector



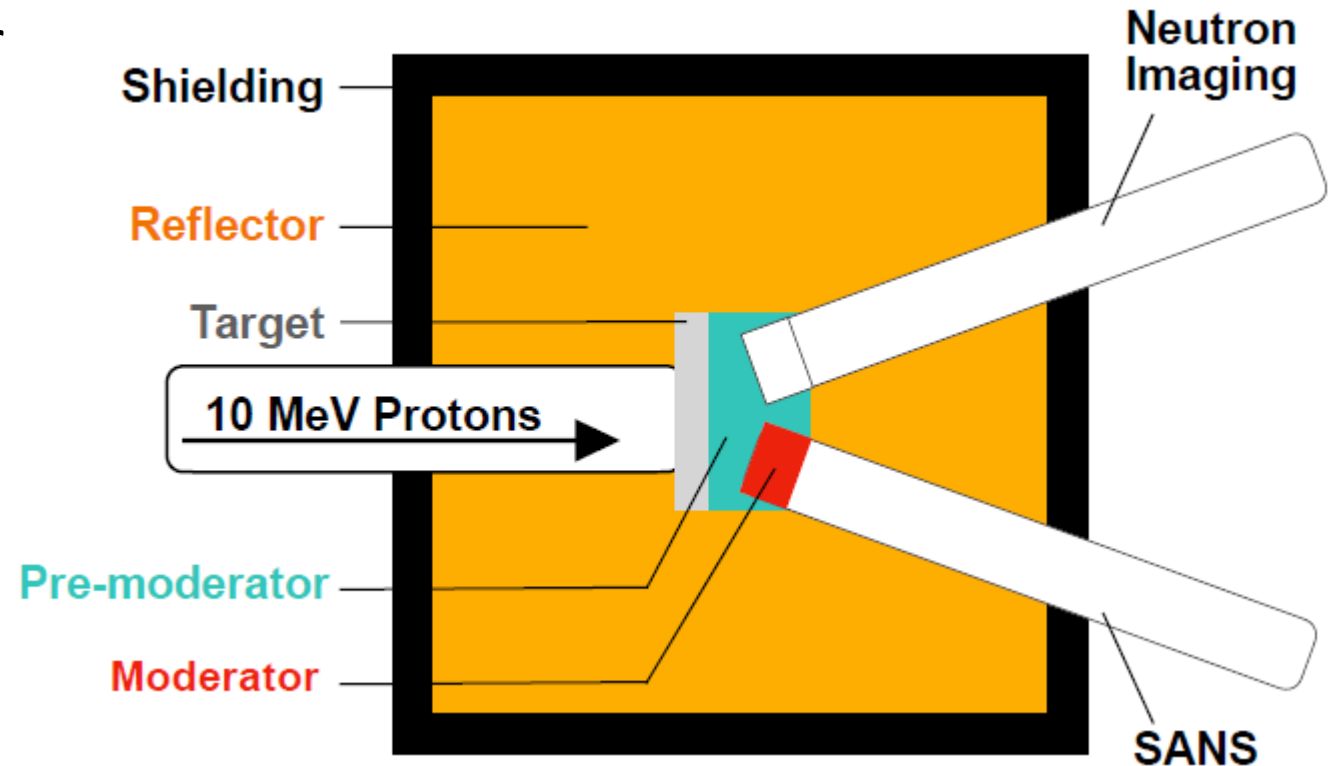
# Summary of TMR Objectives & Funding Prospects

## I. Current Activities in TMR Optimization

- Evaluation of neutron pulse duration for SANS instrument
- Optimize material thicknesses for baseline design for two tube arrangement
- Evaluate SANS instrument performance based on optimized solution

**II. CFI application to be submitted in July 2022**

**III. Conceptual design report to be released in June/July 2022.**





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