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Design, Simulation, and Offline Testing of a Fast Neutron Moderator for AD-BNCT

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abstract: The Cotton University Particle Accelerator Centre and North East (CUPAC-NE)[1] is developing the first particle accelerator facility in Northeast India, a region with the nation's highest cancer incidence. This work focuses on Accelerator-Driven Boron Neutron Capture Therapy (AD-BNCT), an emerging cancer treatment based on the $^{10}\text{B}(n,\alpha)^7\text{Li}$ reaction that selectively destroys boron-loaded tumor cells. Clinical efficacy depends on neutron energy: thermal neutrons (0.025 eV) are suited for superficial tumors, while epithermal neutrons (1 eV - 10 keV) penetrate deeper tissues and subsequently thermalize. Optimized moderation is therefore critical to maximize epithermal flux while ensuring patient safety.

The $p(^7\text{Li},^7\text{Be})n$ reaction[2] at 28 MeV in inverse kinematics is employed at the HIRA facility (IUAC)[3] as the neutron source. Geometric constraints restrict moderator placement to 33° – 50° and ≤ 200 mm to avoid overlap with the first quadrupole (Q1) as shown in Fig.1, thereby fixing the beam energy. The primary neutron branch (1 - 5 MeV) is kinematically correlated with a satellite ^7Be branch (17 - 19 MeV), detectable at HIRA.

As current GEANT4/TOPAS models are unreliable above 23 MeV in inverse kinematics, a transformation code was developed to convert direct to inverse kinematics at equal center-of-mass energy, validated against GEANT4 within its reliable range. Moderator optimization was conducted with CAD-integrated GEANT4 simulations. Conical moderators, commonly employed in AD-BNCT, were found to be inefficient. After several iterations, an optimized design with $14\times$ higher efficiency was achieved as shown in Fig. 2. The system uses water as moderator, Teflon as cone material (chemically inert, providing additional fast-neutron absorption via $^{19}\text{F}(n,\alpha)^{16}\text{O}$ and favorable scattering), and Bi as a gamma shield. Simulations predict $\sim 3 \times 10^4$ thermal neutrons per hour from a 28 MeV, 5 pA ^7Li beam on a $20\ \mu\text{m}$ polypropylene foil, further enhanced by a Pb collimator. The spectrum exhibits a thermal peak at ~ 48 meV with FWHM ~ 147 meV.

A scaled moderator prototype was 3D-printed in PLA (Polylactic Acid), with simulations indicating that a gypsum coating achieves efficiencies comparable to Teflon. A preliminary test is planned at the medical cyclotron of the State Cancer Institute, Guwahati. For thermal neutron detection, a novel and cost-effective method using LR115(II) solid-state nuclear track detectors is proposed. In this method, moderated thermal neutrons interact with ^{10}B compounds, producing α -particles that generate observable tracks under an optical microscope. The technique was validated using a ^{241}Am α -source, which produced well-defined tracks. Using this technique, the thermal neutron flux at the moderator exit can be determined. The medical cyclotron is also equipped with a BF_3 neutron detector, which allows evaluation of the moderator's efficiency by comparing the incident-to-exit flux ratio. Following optimization, the final experiment will be conducted at HIRA (IUAC, AUC:71368), where accelerator-driven monochromatic neutrons will be moderated to produce a well-characterized thermal neutron beam.

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